Lingfeng He

List of Publications by Year
in descending order

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155	3,159	147801	223800
papers	citations	31 h-index	46 g-index

157	157	157	2151
all docs	docs citations	times ranked	citing authors

#	Article	IF	Citations
1	Preparation and strengthening mechanism of prestressed ceramic tile components. International Journal of Applied Ceramic Technology, 2022, 19, 604-611.	2.1	9
2	Acoustic mapping by picosecond ultrasonics for elastic property measurement: Experimental demonstration on a TRISO fuel compact. Journal of Nuclear Materials, 2022, 558, 153391.	2.7	2
3	Visualizing time-dependent microstructural and chemical evolution during molten salt corrosion of Ni-20Cr model alloy using correlative quasi in situ TEM and in situ synchrotron X-ray nano-tomography. Corrosion Science, 2022, 195, 109962.	6.6	19
4	Sensitization, desensitization, and carbide evolution of Alloy 800H made by laser powder bed fusion. Additive Manufacturing, 2022, 50, 102547.	3.0	2
5	Compositionally graded specimen made by laser additive manufacturing as a high-throughput method to study radiation damages and irradiation-assisted stress corrosion cracking. Journal of Nuclear Materials, 2022, 560, 153493.	2.7	9
6	Chemical and elemental mapping of spent nuclear fuel sections by soft X-ray spectromicroscopy. Journal of Synchrotron Radiation, 2022, 29, 67-79.	2.4	3
7	A Square Pulse Thermoreflectance Technique for the Measurement of Thermal Properties. International Journal of Thermophysics, 2022, 43, 1.	2.1	3
8	Raman and photoluminescence evaluation of ion-induced damage uniformity in ThO2. Nuclear Instruments & Methods in Physics Research B, 2022, 515, 69-79.	1.4	4
9	Thermal Energy Transport in Oxide Nuclear Fuel. Chemical Reviews, 2022, 122, 3711-3762.	47.7	37
10	Inferring relative dose-dependent color center populations in proton irradiated thoria single crystals using optical spectroscopy. Physical Chemistry Chemical Physics, 2022, 24, 6133-6145.	2.8	6
11	Dislocation loop evolution in Krâ€irradiated ThO₂. Journal of the American Ceramic Society, 2022, 105, 5419-5435.	3.8	11
12	Unraveling small-scale defects in irradiated ThO2 using kinetic Monte Carlo simulations. Scripta Materialia, 2022, 214, 114684.	5.2	4
13	Phase stability, mechanical properties, and ion irradiation effects in face-centered cubic CrFeMnNi compositionally complex solid-solution alloys at high temperatures. Journal of Nuclear Materials, 2022, 565, 153733.	2.7	8
14	Synthesis and Characterization of Uranium Trichloride in Alkali-Metal Chloride Media. Journal of Nuclear Materials, 2022, , 153728.	2.7	2
15	Impact of small defects and dislocation loops on phonon scattering and thermal transport in <mml:math xmlns:mml="http://www.w3.org/1998/Math/MathML" altimg="si29.svg"><mml:mrow><mml:mtext>Th</mml:mtext><mml:msub><mml:mi mathvariant="normal">O</mml:mi><mml:mn>2</mml:mn></mml:msub></mml:mrow></mml:math>. Journal of Nuclear Materials, 2022, 566, 153758.	2.7	5
16	Thermal conductivity reduction in (Zr_{0.25}Ta_{0.25}Nb_{0.25}Ti_{0.25})C high entropy carbide from extrinsic lattice defects. Materials Research Letters, 2022, 10, 611-617.	8.7	3
17	Measurement of grain boundary strength of Inconel X-750 superalloy using in-situ micro-tensile testing techniques in FIB/SEM system. Materials Science & Engineering A: Structural Materials: Properties, Microstructure and Processing, 2022, 849, 143475.	5.6	2
18	Investigating corrosion behavior of Ni and Ni-20Cr in molten ZnCl2. Corrosion Science, 2021, 179, 109105.	6.6	22

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19	Fuel-cladding chemical interaction of a prototype annular U-10Zr fuel with Fe-12Cr ferritic/martensitic HT-9 cladding. Journal of Nuclear Materials, 2021, 544, 152588.	2.7	17
20	Thermal conductivity of ThO2: Effect of point defect disorder. Journal of Applied Physics, 2021, 129, .	2.5	11
21	Training artificial neural networks for precision orientation and strain mapping using 4D electron diffraction datasets. Ultramicroscopy, 2021, 231, 113256.	1.9	18
22	Indirect characterization of point defects in proton irradiated ceria. Materialia, 2021, 15, 101019.	2.7	16
23	Phase and defect evolution in uranium-nitrogen-oxygen system under irradiation. Acta Materialia, 2021, 208, 116778.	7.9	21
24	Metallic Fast Reactor Separate Effect Studies for Fuel Safety. Journal of Nuclear Engineering and Radiation Science, 2021, 7, .	0.4	0
25	Determining oxidation states of transition metals in molten salt corrosion using electron energy loss spectroscopy. Scripta Materialia, 2021, 197, 113790.	5.2	15
26	Understanding spinodal and binodal phase transformations in U-50Zr. Materialia, 2021, 16, 101092.	2.7	14
27	Isotopic Analysis of Irradiated Ceramic Fuel for Burnup and Microchemical Assessment Using Atom Probe Tomography.. Microscopy and Microanalysis, 2021, 27, 416-417.	0.4	0
28	Machine Learning Based Precision Orientation and Strain Mapping from 4D Diffraction Datasets. Microscopy and Microanalysis, 2021, 27, 1276-1278.	0.4	0
29	An integrated experimental and computational investigation of defect and microstructural effects on thermal transport in thorium dioxide. Acta Materialia, 2021, 213, 116934.	7.9	26
30	Advanced Characterization of Additively Manufactured 316L Stainless Steel for Nuclear Applications. Microscopy and Microanalysis, 2021, 27, 2160-2161.	0.4	0
31	TEM characterization of dislocation loops in proton irradiated single crystal ThO2. Journal of Nuclear Materials, 2021, 552, 152998.	2.7	16
32	In situ monitoring of microstructure evolution during thermal processing of uranium-zirconium alloys using laser-generated ultrasound. Journal of Nuclear Materials, 2021, 553, 153005.	2.7	9
33	First principle studies of effects of solute segregation on grain boundary strength in Ni-based alloys. Journal of Alloys and Compounds, 2021, 874, 159795.	5.5	13
34	Nanoscale redistribution of alloying elements in high-burnup AXIOM-2 (X2Â®) and their effects on in-reactor corrosion. Corrosion Science, 2021, 190, 109652.	6.6	4
35	Effects of heat treatment on corrosion fatigue and stress corrosion crack growth of additive-manufactured Alloy 800H in high-temperature water. Corrosion Science, 2021, 191, 109739.	6.6	9
36	Structure of the pellet-cladding interaction layer of a high-burnup Zr-Nb-O nuclear fuel cladding. Journal of Nuclear Materials, 2021, 556, 153196.	2.7	7

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37	Early-stage microstructural evolution and phase stability in neutron-irradiated ferritic-martensitic steel T91. Journal of Nuclear Materials, 2021, 557, 153207.	2.7	3
38	Phase stability and microstructural evolution in neutron-irradiated ferritic-martensitic steel HT9. Journal of Nuclear Materials, 2021, 557, 153252.	2.7	2
39	Dislocation Loops in Proton Irradiated Uranium-Nitrogen-Oxygen System. Journal of Nuclear Materials, 2021, 557, 153244.	2.7	6
40	The dynamic evolution of swelling in nickel concentrated solid solution alloys through inÂsitu property monitoring. Applied Materials Today, 2021, 25, 101187.	4.3	4
41	A transmission electron microscopy study of EBR-II neutron-irradiated austenitic stainless steel 304 and nickel-base alloy X-750. Journal of Nuclear Materials, 2020, 528, 151851.	2.7	11
42	Experimental evidence for â€˜buckle, ruck and tuckâ€™ in neutron irradiated graphite. Carbon, 2020, 159, 119-121.	10.3	19
43	Listening to Radiation Damage In Situ: Passive and Active Acoustic Techniques. Jom, 2020, 72, 197-209.	1.9	8
44	The influence of lattice defects, recombination, and clustering on thermal transport in single crystal thorium dioxide. APL Materials, 2020, 8, .	5.1	32
45	Impact of krypton irradiation on a single crystal tungsten: Multi-modal X-ray imaging study. Scripta Materialia, 2020, 188, 296-301.	5.2	2
46	Degradation mechanism of lead-vanado-iodoapatite in NaCl solution. Corrosion Science, 2020, 172, 108720.	6.6	3
47	In situ microstructural evolution in face-centered and body-centered cubic complex concentrated solid-solution alloys under heavy ion irradiation. Acta Materialia, 2020, 198, 85-99.	7.9	36
48	Enhanced Resistance to Irradiation Induced Ferritic Transformation in Nanostructured Austenitic Steels. Materialia, 2020, 13, 100806.	2.7	9
49	Î±-U and Ï‰-UZr2 in neutron irradiated U-10Zr annular metallic fuel. Journal of Nuclear Materials, 2020, 542, 152536.	2.7	28
50	Dual Functional Ni₃S₂@Ni Coreâ€“Shell Nanoparticles Decorating Nanoporous Carbon as Cathode Scaffolds for Lithiumâ€“Sulfur Battery with Lean Electrolytes. ACS Applied Energy Materials, 2020, 3, 4173-4179.	5.1	19
51	Fullerene-like defects in high-temperature neutron-irradiated nuclear graphite. Carbon, 2020, 166, 113-122.	10.3	20
52	Combining mesoscale thermal transport and x-ray diffraction measurements to characterize early-stage evolution of irradiation-induced defects in ceramics. Acta Materialia, 2020, 193, 61-70.	7.9	25
53	Effect of proton pre-irradiation on corrosion of Zr-0.5Nb model alloys with different Nb distributions. Corrosion Science, 2020, 173, 108790.	6.6	8
54	Non-contact, non-destructive mapping of thermal diffusivity and surface acoustic wave speed using transient grating spectroscopy. Review of Scientific Instruments, 2020, 91, 054902.	1.3	13

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55	Development of a grain growth model for U3Si2 using experimental data, phase field simulation and molecular dynamics. Journal of Nuclear Materials, 2020, 532, 152069.	2.7	13
56	Revealing 3D Morphological and Chemical Evolution Mechanisms of Metals in Molten Salt by Multimodal Microscopy. ACS Applied Materials & Interfaces, 2020, 12, 17321-17333.	8.0	20
57	On spinodal-like phase decomposition in Uâ€“50Zr alloy. Materialia, 2020, 9, 100592.	2.7	20
58	Electron microscopy characterization of fast reactor MOX Joint Oxyde-Gaine (JOG). Journal of Nuclear Materials, 2020, 531, 151964.	2.7	13
59	The correlation between microstructure and nanoindentation property of neutron-irradiated austenitic alloy D9. Acta Materialia, 2020, 195, 433-445.	7.9	12
60	Intragranular thermal transport in Uâ€“50Zr. Journal of Nuclear Materials, 2020, 534, 152145.	2.7	9
61	Atom Probe Tomography for Burnup and Fission Product Analysis for Nuclear Fuels. Microscopy and Microanalysis, 2020, 26, 3086-3088.	0.4	1
62	Investigating the Effect of CrCl3 on Corrosion Behavior of Ni and Ni-20Cr in Molten ZnCl2 Salt By Electrochemical Noise Measurements. ECS Meeting Abstracts, 2020, MA2020-02, 2923-2923.	0.0	0
63	4D in Situ Temperature-Dependent Study on Morphological and Chemical Evolution of Metals in Molten Salt Environments By Multimodal Microscopy. ECS Meeting Abstracts, 2020, MA2020-02, 2925-2925.	0.0	0
64	Diffusion behaviors between metallic fuel alloys with Pd addition and Fe. Journal of Nuclear Materials, 2019, 525, 111-124.	2.7	10
65	A simple way to make pre-stressed ceramics with high strength. Journal of Materiomics, 2019, 5, 657-662.	5.7	16
66	Microstructure and microchemistry study of irradiation-induced precipitates in proton irradiated ZrNb alloys. Acta Materialia, 2019, 178, 228-240.	7.9	33
67	Inferring radiation-induced microstructural evolution in single-crystal niobium through changes in thermal transport. Journal of Nuclear Materials, 2019, 523, 378-382.	2.7	17
68	Impact of irradiation induced dislocation loops on thermal conductivity in ceramics. Journal of the American Ceramic Society, 2019, 102, 7533-7542.	3.8	56
69	Microstructural characterization of annealed U-20Pu-10Zr-3.86Pd and U-20Pu-10Zr-3.86Pd-4.3Ln. Journal of Nuclear Materials, 2019, 518, 287-297.	2.7	9
70	Transient grating spectroscopy: An ultrarapid, nondestructive materials evaluation technique. MRS Bulletin, 2019, 44, 392-402.	3.5	37
71	Self-healing behavior and strength recovery of ytterbium disilicate ceramic reinforced with silicon carbide nanofillers. Journal of the European Ceramic Society, 2019, 39, 3139-3152.	5.7	26
72	Real-time thermomechanical property monitoring during ion beam irradiation using in situ transient grating spectroscopy. Nuclear Instruments & Methods in Physics Research B, 2019, 440, 126-138.	1.4	27

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73	Formation of tetragonal gas bubble superlattice in bulk molybdenum under helium ion implantation. Scripta Materialia, 2018, 149, 26-30.	5.2	12
74	Interdiffusion behavior of U3Si2 with FeCrAl via diffusion couple studies. Journal of Nuclear Materials, 2018, 502, 356-369.	2.7	10
75	Strengthening of a lithium disilicate glass-ceramic by rapid cooling. Ceramics International, 2018, 44, 11650-11657.	4.8	21
76	Radiation-induced grain subdivision and bubble formation in U3Si2 at LWR temperature. Journal of Nuclear Materials, 2018, 498, 169-175.	2.7	25
77	Hydrothermal synthesis of silicon oxide clad uranium oxide nanowires. Journal of the American Ceramic Society, 2018, 101, 1004-1008.	3.8	3
78	Challenges and Opportunities on Elucidating Irradiated Fuels with Atom Probe Tomography. Microscopy and Microanalysis, 2018, 24, 2206-2207.	0.4	0
79	STEM-EDS/EELS and APT characterization of ZrN coatings on UMo fuel kernels. Journal of Nuclear Materials, 2018, 511, 174-182.	2.7	12
80	In-situ TEM study of the ion irradiation behavior of U3Si2 and U3Si5. Journal of Nuclear Materials, 2018, 511, 56-63.	2.7	12
81	Thermal stability of helium bubble superlattice in Mo under TEM in-situ heating. Journal of Nuclear Materials, 2018, 505, 207-211.	2.7	6
82	Erratum to Interdiffusion Behavior of U3Si2 with FeCrAl via Diffusion Couple Studies J. Nucl. Mater. 502 (2018) 356â€“369. Journal of Nuclear Materials, 2018, 507, 403-417.	2.7	0
83	Microstructural characterization of as-cast U-20Pu-10Zr-3.86Pd and U-20Pu-10Zr-3.86Pd-4.3Ln. Journal of Nuclear Materials, 2018, 508, 310-318.	2.7	11
84	Microstructural characterization of annealed U-12Zr-4Pd and U-12Zr-4Pd-5Ln: Investigating Pd as a metallic fuel additive. Journal of Nuclear Materials, 2018, 502, 106-112.	2.7	23
85	Interdiffusion Behavior of FeCrAl with U3Si2. Minerals, Metals and Materials Series, 2018, , 175-184.	0.4	0
86	Strength improvement and purification of Yb₂Si₂O₇â€“SiC nanocomposites by surface oxidation treatment. Journal of the American Ceramic Society, 2017, 100, 3122-3131.	3.8	18
87	Microstructure studies of interdiffusion behavior of U3Si2/Zircaloy-4 at 800 and 1000ÂÂ°C. Journal of Nuclear Materials, 2017, 486, 274-282.	2.7	17
88	Effects of neutron irradiation of Ti3SiC2 and Ti3AlC2 in the 121â€“1085ÂÂ°C temperature range. Journal of Nuclear Materials, 2017, 484, 120-134.	2.7	63
89	Bubble evolution in Kr-irradiated UO2 during annealing. Journal of Nuclear Materials, 2017, 496, 242-250.	2.7	13
90	Irradiation effects in Generation IV nuclear reactor materials. , 2017, , 253-283.		12

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91	Bridging the gap to mesoscale radiation materials science with transient grating spectroscopy. Physical Review B, 2016, 94, .	3.2	26
92	Corrosion-induced microstructural developments in 316 stainless steel during exposure to molten Li2BeF4(FLiBe) salt. Journal of Nuclear Materials, 2016, 482, 147-155.	2.7	36
93	Subsurface imaging of grain microstructure using picosecond ultrasonics. Acta Materialia, 2016, 112, 209-215.	7.9	26
94	Effect of neutron irradiation on defect evolution in Ti3SiC2 and Ti2AlC. Journal of Nuclear Materials, 2016, 468, 194-206.	2.7	65
95	Inert Gas Measurement of Single Bubble in CeO2. Microscopy and Microanalysis, 2015, 21, 751-752.	0.4	1
96	Annealing-induced lattice recovery in room-temperature xenon irradiated CeO₂: X-ray diffraction and electron energy loss spectroscopy experiments. Journal of Materials Research, 2015, 30, 1555-1562.	2.6	7
97	Fluctuation Electron Microscopy Study of Medium-Range Packing Order in Ultrastable Indomethacin Glass Thin Films. Materials Research Society Symposia Proceedings, 2015, 1757, 32.	0.1	0
98	Applications of Transient Grating Spectroscopy to Radiation Materials Science. Jom, 2015, 67, 1840-1848.	1.9	18
99	High-Temperature Corrosion of UNS N10003 in Molten Li₂BeF₄(FLiBe) Salt. Corrosion, 2015, 71, 1257-1266.	1.1	33
100	Bubble formation and Kr distribution in Kr-irradiated UO2. Journal of Nuclear Materials, 2015, 456, 125-132.	2.7	29
101	Effect of Grain Boundaries on Krypton Segregation Behavior in Irradiated Uranium Dioxide. Jom, 2014, 66, 2562-2568.	1.9	7
102	Corrosion behavior of an alumina forming austenitic steel exposed to supercritical carbon dioxide. Corrosion Science, 2014, 82, 67-76.	6.6	79
103	Thermal Conductivity in Nanocrystalline Ceria Thin Films. Journal of the American Ceramic Society, 2014, 97, 562-569.	3.8	58
104	Synthesis of BN nanosheet/nanotube-Fe nanocomposites by pulsed wire discharge and high-temperature annealing. Materials Letters, 2014, 117, 120-123.	2.6	12
105	In Situ TEM Observation of Dislocation Evolution in Polycrystalline UO2. Jom, 2014, 66, 2553-2561.	1.9	17
106	Microstructure changes and thermal conductivity reduction in UO2 following 3.9 MeV He2+ ion irradiation. Journal of Nuclear Materials, 2014, 454, 283-289.	2.7	38
107	Microstructure evolution in Xe-irradiated UO2 at room temperature. Nuclear Instruments & Methods in Physics Research B, 2014, 330, 55-60.	1.4	31
108	2.6MeV proton irradiation effects on the surface integrity of depleted UO2. Nuclear Instruments & Methods in Physics Research B, 2014, 319, 100-106.	1.4	15

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109	Synthesis and structureâ€“property relationships of a new family of layered carbides in Zr-Al(Si)-C and Hf-Al(Si)-C systems. Journal of the European Ceramic Society, 2013, 33, 2831-2865.	5.7	89
110	In situ TEM observation of dislocation evolution in Kr-irradiated UO2 single crystal. Journal of Nuclear Materials, 2013, 443, 71-77.	2.7	51
111	Transmission Electron Microscopy Investigation of Krypton Bubbles in Polycrystalline CeO₂. Nuclear Technology, 2013, 182, 164-169.	1.2	8
112	Fission Products in Nuclear Fuel: Comparison of Simulated Distribution with Correlative Characterization Techniques. Microscopy and Microanalysis, 2013, 19, 968-969.	0.4	3
113	Fabrication and Characterization of Tricalcium Silicate Bioceramics with High Mechanical Properties by Spark Plasma Sintering. International Journal of Applied Ceramic Technology, 2011, 8, 501-510.	2.1	9
114	Effect of Ti Dopant on the Mechanical Properties and Oxidation Behavior of Zr2[Al(Si)]4C5 Ceramics. Journal of the American Ceramic Society, 2011, 94, 1872-1877.	3.8	12
115	Mechanical properties of Y2Ti2O7. Scripta Materialia, 2011, 64, 548-551.	5.2	42
116	Mechanical properties and bioactivity of Î²-Ca2SiO4 ceramics synthesized by spark plasma sintering. Ceramics International, 2011, 37, 2459-2465.	4.8	41
117	Mechanical and thermal properties of a Hf2[Al(Si)]4C5 ceramic prepared by in situ reaction/hot-pressing. Scripta Materialia, 2010, 62, 427-430.	5.2	5
118	Surface strengthening of Ti3SiC2 through magnetron sputtering of Mo and Zr and subsequent annealing. Journal of the European Ceramic Society, 2010, 30, 2123-2130.	5.7	1
119	Microstructure, mechanical, thermal, and oxidation properties of a Zr2[Al(Si)]4C5â€“SiC composite prepared by in situ reaction/hot-pressing. Journal of the European Ceramic Society, 2010, 30, 2147-2154.	5.7	18
120	In situ Reaction Synthesis and Mechanical Properties of TaCâ€“TaSi₂ Composites. International Journal of Applied Ceramic Technology, 2010, 7, 697-703.	2.1	14
121	Crystal Structure and Theoretical Elastic Property of a New Ternary Ceramic HfAl₄C₄. Journal of the American Ceramic Society, 2010, 93, 1164-1168.	3.8	9
122	Mechanisms and Kinetics of the Hydrothermal Oxidation of Bulk Titanium Silicon Carbide. Journal of the American Ceramic Society, 2010, 93, 1148-1155.	3.8	24
123	A New Method to Improve the Highâ€Temperature Mechanical Properties of Ti₃SiC₂by Substituting Ti with Zr, Hf, or Nb. Journal of the American Ceramic Society, 2010, 93, 1749-1753.	3.8	30
124	Reciprocating Friction and Wear Behavior of Zr₂[Al(Si)]₄C₅ and Zr₂[Al(Si)]₄C₅â€“SiC Composite Against Si₃N₄ Ball. Journal of the American Ceramic Society, 2010, 93, 2369-2376.	3.8	11
125	Oxidation Behavior of Ternary Carbide Ceramics in Hfâ€“Alâ€“C System in Air. Journal of the American Ceramic Society, 2010, 93, 3427-3431.	3.8	10
126	Ultrahigh-temperature oxidation of Zr2Al3C4 via rapid induction heating. Scripta Materialia, 2009, 60, 547-550.	5.2	29

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127	Highly conductive and strengthened copper matrix composite reinforced by Zr2Al3C4 particulates. Scripta Materialia, 2009, 60, 976-979.	5.2	48
128	High-temperature internal friction, stiffness and strength of Zrâ€“Al(Si)â€“C ceramics. Scripta Materialia, 2009, 61, 60-63.	5.2	39
129	Tribological Properties of a Zr₂Al₃C₄Ceramic at Ambient Temperature. Journal of the American Ceramic Society, 2009, 92, 141-146.	3.8	5
130	Mechanical and Thermophysical Properties of Zrâ€“Alâ€“Siâ€“C Ceramics. Journal of the American Ceramic Society, 2009, 92, 445-451.	3.8	40
131	Zirconium Aluminum Carbides: New Precursors for Synthesizing ZrO₂â€“Al₂O₃ Composites. Journal of the American Ceramic Society, 2009, 92, 2751-2758.	3.8	8
132	Microstructure and mechanical and thermal properties of ternary carbides in Hfâ€“Alâ€“C system. Acta Materialia, 2009, 57, 2765-2774.	7.9	33
133	Microstructure and properties of bulk Ta2AlC ceramic synthesized by an in situ reaction/hot pressing method. Journal of the European Ceramic Society, 2008, 28, 1679-1685.	5.7	63
134	Atomic-scale microstructure and elastic properties of quaternary Zrâ€“Alâ€“Siâ€“C ceramics. Acta Materialia, 2008, 56, 2022-2031.	7.9	54
135	In Situ Reaction Synthesis, Electrical and Thermal, and Mechanical Properties of Nb₄AlC₃. Journal of the American Ceramic Society, 2008, 91, 2258-2263.	3.8	112
136	Synthesis, Microstructure, and Mechanical Properties of Al₃BC₃. Journal of the American Ceramic Society, 2008, 91, 2343-2348.	3.8	16
137	In Situ Reaction Synthesis and Mechanical Properties of V₂AlC. Journal of the American Ceramic Society, 2008, 91, 4029-4035.	3.8	78
138	Crystal structure and theoretical elastic property of two new ternary ceramics Hf3Al4C6 and Hf2Al4C5. Scripta Materialia, 2008, 58, 679-682.	5.2	46
139	Oxidation of Zr₂[Al(Si)]₄C₅ and Zr₃[Al(Si)]₄C₆ in air. Journal of Materials Research, 2008, 23, 3339-3346.	2.6	41
140	Isothermal oxidation of bulk Zr₂Al₃C₄ at 500 to 1000 Â°C in air. Journal of Materials Research, 2008, 23, 359-366.	2.6	39
141	Improving the high-temperature oxidation resistance of Zr2Al3C4 by silicon pack cementation. Journal of Materials Research, 2008, 23, 2275-2282.	2.6	14
142	Elastic and thermal properties of Zr2Al3C4: Experimental investigations and ab initio calculations. Journal of Applied Physics, 2007, 102, .	2.5	32
143	Low energy ion assisted deposition of Taâ•Cu films. Journal of Applied Physics, 2007, 101, 024318.	2.5	12
144	Layered stacking characteristics of ternary zirconium aluminum carbides. Journal of Materials Research, 2007, 22, 3058-3066.	2.6	31

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145	Synthesis, Physical, and Mechanical Properties of Bulk Zr3Al3C5Ceramic. Journal of the American Ceramic Society, 2007, 90, 1164-1170.	3.8	62
146	Physical and Mechanical Properties of Bulk Ta4AlC3Ceramic Prepared by an In Situ Reaction Synthesis/Hot-Pressing Method. Journal of the American Ceramic Society, 2007, 90, 2542-2548.	3.8	82
147	In SituSynthesis and Properties of Ti₃AlC₂/TiB₂Composites. Journal of the American Ceramic Society, 2007, 90, 3615-3620.	3.8	27
148	Synthesis and Characterization of Bulk Zr₂Al₃C₄Ceramic. Journal of the American Ceramic Society, 2007, 90, 3687-3689.	3.8	67
149	Atomic-scale microstructures of Zr2Al3C4 and Zr3Al3C5 ceramics. Acta Materialia, 2006, 54, 3843-3851.	7.9	63
150	Effect of thin strain-compensated Al0.6Ga0.4P layers on the growth of multiple-stacked InP/In0.5Al0.3Ga0.2P quantum dots. Journal of Electronic Materials, 2006, 35, 701-704.	2.2	2
151	First-principles prediction of the mechanical properties and electronic structure of ternary aluminum carbideZr3Al3C5. Physical Review B, 2006, 73, .	3.2	67
152	On Spinodal-Like Phase Decomposition in U-50 Zr Alloy. SSRN Electronic Journal, 0, , .	0.4	0
153	Fuel-Cladding Chemical Interaction of a Prototype Annular U-10Zr Fuel with Fe-12Cr Ferritic/Martensitic Cladding. SSRN Electronic Journal, 0, , .	0.4	0
154	In Situ Microstructural Evolution in Fcc and Bcc Complex Concentrated Solid-Solution Alloys Under Heavy Ion Irradiation. SSRN Electronic Journal, 0, , .	0.4	0
155	Proton irradiation-induced blistering in UO2. MRS Advances, 0, , 1.	0.9	1