Rajesh Maingi

List of Publications by Year
in descending order

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274 papers	9,828 citations	31976 53 h-index	56724 83 g-index
280 all docs	280 docs citations	280 times ranked	2806 citing authors

#	Article	IF	Citations
1	Fusion pilot plant performance and the role of a sustained high power density tokamak. Nuclear Fusion, 2022, 62, 036026.	3.5	13
2	EAST steady-state long pulse H-mode with core-edge integration for CFETR. Nuclear Fusion, 2022, 62, 076009.	3.5	14
3	Impact of edge harmonic oscillations on the divertor heat flux in NSTX. Physics of Plasmas, 2022, 29, 012503.	1.9	1
4	NSTX-U theory, modeling and analysis results. Nuclear Fusion, 2022, 62, 042023.	3.5	8
5	Critical role of current-driven instabilities for ELMs in NSTX. Nuclear Fusion, 2022, 62, 076018.	3.5	1
6	Corrosion characteristics of Mo and TZM alloy for plasma facing components in molten lithium at 623ÂK. Corrosion Science, 2022, 200, 110202.	6.6	14
7	Simulation of Liquid Lithium Divertor Geometry Using SOLPS-ITER. IEEE Transactions on Plasma Science, 2022, , 1-7.	1.3	6
8	Numerical Analysis of Liquid Metal MHD Flow and Heat Transfer for Open-Surface Li Divertor in FNSF. IEEE Transactions on Plasma Science, 2022, 50, 4193-4198.	1.3	2
9	Modeling of liquid lithium flow in porous plasma facing material. Nuclear Materials and Energy, 2021, 26, 100935.	1.3	8
10	3D modeling of boron transport in DIII-D L-mode wall conditioning experiments. Nuclear Materials and Energy, 2021, 26, 100900.	1.3	10
11	Progress of Divertor Heat and Particle Flux Control in EAST for Advanced Steady-State Operation in the Last 10 Years. Journal of Fusion Energy, 2021, 40, 1.	1.2	9
12	Type-I ELM mitigation by continuous lithium granule gravitational injection into the upper tungsten divertor in EAST. Nuclear Fusion, 2021, 61, 066022.	3.5	4
13	Enhancement of edge turbulence concomitant with ELM suppression during boron powder injection in EAST. Physics of Plasmas, 2021, 28, 082512.	1.9	4
14	Comparison of active impurity control between lithium and boron powder real-time injection in EAST. Physica Scripta, 2021, 96, 124034.	2.5	4
15	Wall conditioning and ELM mitigation with boron nitride powder injection in KSTAR. Nuclear Materials and Energy, 2021, 28, 101043.	1.3	12
16	Suppression of edge localized modes with real-time boron injection using the tungsten divertor in EAST. Nuclear Fusion, 2021, 61, 014002.	3.5	33
17	A Domestic Program for Liquid Metal PFC Research in Fusion. Journal of Fusion Energy, 2020, 39, 441-447.	1.2	9
18	Effect of lithium coating on long pulse high performance plasma discharges in EAST. Plasma Physics and Controlled Fusion, 2020, 62, 085012.	2.1	14

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19	Lithium splashing from flowing liquid lithium limiter and its effect on high confinement plasma performance in EAST tokamak. Nuclear Materials and Energy, 2020, 25, 100845.	1.3	3
20	Enhanced pedestal H-mode at low edge ion collisionality on NSTX. Physics of Plasmas, 2020, 27, 072511.	1.9	8
21	ELM Suppression by Boron Powder Injection and Comparison with Lithium Powder Injection on EAST. Journal of Fusion Energy, 2020, 39, 429-435.	1.2	14
22	Results from a new flowing liquid Li limiter with TZM substrate during high confinement plasmas in the EAST device. Physics of Plasmas, 2020, 27, .	1.9	10
23	Development of a new TZM substrate flowing liquid lithium limiter for high performance plasma discharge in EAST. Fusion Engineering and Design, 2020, 158, 111747.	1.9	6
24	Overview of lithium injection and flowing liquid lithium results from the USâ€“China collaboration on EAST. Physica Scripta, 2020, T171, 014067.	2.5	8
25	Perspectives on the FESAC transformative enabling capabilities: Priorities, plans, and Status. Fusion Engineering and Design, 2020, 155, 111529.	1.9	1
26	Real-time gas cooling of flowing liquid lithium limiter for the EAST. Fusion Engineering and Design, 2020, 154, 111537.	1.9	3
27	Innovative approaches towards an economic fusion reactor. National Science Review, 2020, 7, 245-247.	9.5	2
28	Observations of wall conditioning by means of boron powder injection in DIII-D H-mode plasmas. Nuclear Fusion, 2020, 60, 126010.	3.5	27
29	Overview of physics studies on ASDEX Upgrade. Nuclear Fusion, 2019, 59, 112014.	3.5	38
30	Study of the Impact of Pre- and Real-Time Depositions of Lithium on Plasma Performance on NSTX. IEEE Transactions on Plasma Science, 2019, 47, 4225-4232.	1.3	1
31	Integrated operation of steady-state long-pulse H-mode in Experimental Advanced Superconducting Tokamak. Nuclear Fusion, 2019, 59, 086030.	3.5	68
32	Active conditioning of ASDEX Upgrade tungsten plasma-facing components and discharge enhancement through boron and boron nitride particulate injection. Nuclear Fusion, 2019, 59, 126034.	3.5	31
33	Multiple striated heat fluxes patterns on the EAST first wall generated by lower hybrid wave absorption in the scrape-off layer. Physics of Plasmas, 2019, 26, 072506.	1.9	3
34	Promising High-Confinement Regime for Steady-State Fusion. Physical Review Letters, 2019, 122, 255001.	7.8	43
35	Deuterium pellet fueling in type-III ELMy H-mode plasmas on EAST superconducting tokamak. Fusion Engineering and Design, 2019, 145, 79-86.	1.9	5
36	Ablation of solid pellets induced by supra-thermal ions in the far scrape-off layer of DIII-D plasmas. Nuclear Fusion, 2019, 59, 084003.	3.5	6

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37	Supplemental ELM control in ITER through beryllium granule injection. Nuclear Materials and Energy, 2019, 19, 34-41.	1.3	10
38	NSTX/NSTX-U theory, modeling and analysis results. Nuclear Fusion, 2019, 59, 112007.	3.5	20
39	Real-time wall conditioning by controlled injection of boron and boron nitride powder in full tungsten wall ASDEX Upgrade. Nuclear Materials and Energy, 2019, 19, 384-389.	1.3	35
40	Real time wall conditioning with lithium powder injection in long pulse H-mode plasmas in EAST with tungsten divertor. Nuclear Materials and Energy, 2019, 19, 124-130.	1.3	25
41	Summary of the FESAC Transformative Enabling Capabilities Panel Report. Fusion Science and Technology, 2019, 75, 167-177.	1.1	7
42	Design of the Flowing LIquid Torus (FLIT). Nuclear Materials and Energy, 2019, 19, 524-530.	1.3	8
43	A horizontal powder injector for W7-X. Fusion Engineering and Design, 2019, 146, 1403-1407.	1.9	8
44	Results from an improved flowing liquid lithium limiter with increased flow uniformity in high power plasmas in EAST. Nuclear Fusion, 2019, 59, 016009.	3.5	33
45	Experiments of continuously and stably flowing lithium limiter in EAST towards a solution for the power exhaust of future fusion devices. Nuclear Materials and Energy, 2019, 18, 99-104.	1.3	19
46	Active Recycling Control Through Lithium Injection in EAST. IEEE Transactions on Plasma Science, 2018, 46, 1081-1085.	1.3	11
47	Injected mass deposition thresholds for lithium granule instigated triggering of edge localized modes on EAST. Nuclear Fusion, 2018, 58, 036007.	3.5	20
48	First Results of ELM Triggering With a Multichamber Lithium Granule Injector Into EAST Discharges. IEEE Transactions on Plasma Science, 2018, 46, 1076-1080.	1.3	11
49	ELM elimination with Li powder injection in EAST discharges using the tungsten upper divertor. Nuclear Fusion, 2018, 58, 024003.	3.5	45
50	A multi-species powder dropper for magnetic fusion applications. Review of Scientific Instruments, 2018, 89, 10K121.	1.3	40
51	ELM frequency enhancement and discharge modification through lithium granule injection into EAST H-modes. Nuclear Fusion, 2018, 58, 126021.	3.5	8
52	Real-time reduction of tungsten impurity influx using lithium powder injection in EAST. Fusion Engineering and Design, 2018, 137, 202-208.	1.9	16
53	Initial results and designs of dual-filter and plenoptic imaging for high-temperature plasmas. Review of Scientific Instruments, 2018, 89, 10E112.	1.3	4
54	Reduction of hydrogen content in deuterium plasma with mixed graphite and tungsten divertors in EAST. Fusion Engineering and Design, 2018, 131, 41-48.	1.9	5

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55	Latest Results From the Hybrid Illinois Device for Research and Applications (HIDRA). IEEE Transactions on Plasma Science, 2018, 46, 2685-2690.	1.3	13
56	A Quasi-Periodic Linear Feeder for the Impurity Granular Injection on DIII-D. IEEE Transactions on Plasma Science, 2018, 46, 1120-1126.	1.3	4
57	Kinetic simulations of scrape-off layer physics in the DIII-D tokamak. Nuclear Materials and Energy, 2017, 12, 978-983.	1.3	10
58	Stabilizing effects of enhanced resistivity due to lithium-conditioning on low-n edge localized modes in NSTX. Physics of Plasmas, 2017, 24, 054501.	1.9	3
59	Compatibility of lithium plasma-facing surfaces with high edge temperatures in the Lithium Tokamak Experiment. Physics of Plasmas, 2017, 24, .	1.9	28
60	Overview of NSTX Upgrade initial results and modelling highlights. Nuclear Fusion, 2017, 57, 102006.	3.5	45
61	Effect of heating scheme on SOL width in DIII-D and EAST. Nuclear Materials and Energy, 2017, 12, 221-226.	1.3	20
62	Gyrokinetic projection of the divertor heat-flux width from present tokamaks to ITER. Nuclear Fusion, 2017, 57, 116023.	3.5	125
63	Liquid lithium loop system to solve challenging technology issues for fusion power plant. Nuclear Fusion, 2017, 57, 116056.	3.5	19
64	Impact of wall materials and seeding gases on the pedestal and on core plasma performance. Nuclear Materials and Energy, 2017, 12, 18-27.	1.3	36
65	Mitigation of divertor heat flux by high-frequency ELM pacing with non-fuel pellet injection in DIII-D. Nuclear Materials and Energy, 2017, 12, 1030-1036.	1.3	15
66	Modeling of lithium granule injection in NSTX with M3D-C1. Nuclear Materials and Energy, 2017, 12, 1094-1099.	1.3	3
67	Elimination of inter-discharge helium glow discharge cleaning with lithium evaporation in NSTX. Nuclear Materials and Energy, 2017, 12, 720-724.	1.3	1
68	Effect of progressively increasing lithium conditioning on edge transport and stability in high triangularity NSTX H-modes. Fusion Engineering and Design, 2017, 117, 150-156.	1.9	5
69	Upgraded flowing liquid lithium limiter for improving Li coverage uniformity and erosion resistance in EAST device. Review of Scientific Instruments, 2017, 88, 123506.	1.3	16
70	ELM-free and inter-ELM divertor heat flux broadening induced by edge harmonics oscillation in NSTX. Nuclear Fusion, 2017, 57, 126053.	3.5	5
71	Linear gyrokinetic simulations of microinstabilities within the pedestal region of H-mode NSTX discharges in a highly shaped geometry. Physics of Plasmas, 2016, 23, 062520.	1.9	12
72	First results of the use of a continuously flowing lithium limiter in high performance discharges in the EAST device. Nuclear Fusion, 2016, 56, 046011.	3.5	57

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73	Snowflake Divertor Experiments in the DIII-D, NSTX, and NSTX-U Tokamaks Aimed at the Development of the Divertor Power Exhaust Solution. IEEE Transactions on Plasma Science, 2016, 44, 3445-3455.	1.3	14
74	Conference Report on the 4rd International Symposium on Lithium Applications. Nuclear Fusion, 2016, 56, 127002.	3.5	16
75	High frequency pacing of edge localized modes by injection of lithium granules in DIII-D H-mode discharges. Nuclear Fusion, 2016, 56, 056008.	3.5	42
76	Fusion nuclear science facilities and pilot plants based on the spherical tokamak. Nuclear Fusion, 2016, 56, 106023.	3.5	119
77	Lithium granule ablation and penetration during ELM pacing experiments at DIII-D. Fusion Engineering and Design, 2016, 112, 621-627.	1.9	10
78	Dependence of recycling and edge profiles on lithium evaporation in high triangularity, high performance NSTX H-mode discharges. Journal of Nuclear Materials, 2015, 463, 1134-1137.	2.7	17
79	Effect of lithium in the DIII-D SOL and plasma-facing surfaces. Journal of Nuclear Materials, 2015, 463, 1160-1164.	2.7	3
80	Developing snowflake divertor physics basis in the DIII-D, NSTX and NSTX-U tokamaks aimed at the divertor power exhaust solution.. , 2015, , .		0
81	ELM mitigation with pellet ELM triggering and implications for PFCs and plasma performance in ITER. Journal of Nuclear Materials, 2015, 463, 104-108.	2.7	12
82	Application of the radiating divertor approach to innovative tokamak divertor concepts. Journal of Nuclear Materials, 2015, 463, 1225-1228.	2.7	18
83	Heat flux management via advanced magnetic divertor configurations and divertor detachment. Journal of Nuclear Materials, 2015, 463, 1186-1190.	2.7	30
84	Impact of ELM filaments on divertor heat flux dynamics in NSTX. Journal of Nuclear Materials, 2015, 463, 701-704.	2.7	2
85	Lithium as a plasma facing component to optimize the edge plasma. , 2015, , .		0
86	Lithium granular injector operational experience triggering ELMs in H-mode on DIII-D. , 2015, , .		2
87	Control of high-Z PFC erosion by local gas injection in DIII-D. Journal of Nuclear Materials, 2015, 463, 605-610.	2.7	9
88	New Steady-State Quiescent High-Confinement Plasma in an Experimental Advanced Superconducting Tokamak. Physical Review Letters, 2015, 114, 055001.	7.8	93
89	of Plasmas, 2015, 22, 056112.	1.9	31
90	Enhanced H-mode pedestals with lithium injection in DIII-D. Nuclear Fusion, 2015, 55, 063018.	3.5	123

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91	An overview of recent physics results from NSTX. Nuclear Fusion, 2015, 55, 104002.	3.5	21
92	Active control of divertor heat and particle fluxes in EAST towards advanced steady state operations. Journal of Nuclear Materials, 2015, 463, 99-103.	2.7	5
93	Progress toward commissioning and plasma operation in NSTX-U. Nuclear Fusion, 2015, 55, 073007.	3.5	16
94	Broadening of divertor heat flux profile with increasing number of ELM filaments in NSTX. Nuclear Fusion, 2014, 54, 122004.	3.5	6
95	SPIRAL field mapping on NSTX for comparison to divertor RF heat deposition. , 2014, , .		2
96	Towards identifying the mechanisms underlying field-aligned edge-loss of HHFW power on NSTX. , 2014, , .		3
97	Enhanced confinement scenarios without large edge localized modes in tokamaks: control, performance, and extrapolability issues for ITER. Nuclear Fusion, 2014, 54, 114016.	3.5	39
98	Observation of EHO in NSTX and theoretical study of its active control using HHFW antenna. Nuclear Fusion, 2014, 54, 043013.	3.5	4
99	Progress in understanding the enhanced pedestal H-mode in NSTX. Nuclear Fusion, 2014, 54, 083021.	3.5	12
100	The effects of increasing lithium deposition on the power exhaust channel in NSTX. Nuclear Fusion, 2014, 54, 023001.	3.5	15
101	Recent advances in long-pulse high-confinement plasma operations in Experimental Advanced Superconducting Tokamak. Physics of Plasmas, 2014, 21, 056107.	1.9	25
102	Reduced model prediction of electron temperature profiles in microtearing-dominated National Spherical Torus eXperiment plasmas. Physics of Plasmas, 2014, 21, 082510.	1.9	11
103	Feasibility of Power and Particle Handling in an ST-FNSF and the Effects of Divertor Geometry. IEEE Transactions on Plasma Science, 2014, 42, 573-579.	1.3	7
104	Approaches towards long-pulse divertor operations on EAST by active control of plasmaâ€“wall interactions. Nuclear Fusion, 2014, 54, 013002.	3.5	54
105	Advanced divertor configurations with large flux expansion. Journal of Nuclear Materials, 2013, 438, S96-S101.	2.7	24
106	A long-pulse high-confinement plasma regime in the Experimental Advanced Superconducting Tokamak. Nature Physics, 2013, 9, 817-821.	16.7	234
107	Edge microstability of NSTX plasmas without and with lithium-coated plasma-facing components. Nuclear Fusion, 2013, 53, 113016.	3.5	52
108	Overview of physics results from the conclusive operation of the National Spherical Torus Experiment. Nuclear Fusion, 2013, 53, 104007.	3.5	53

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109	Varying the pre-discharge lithium wall coatings to alter the characteristics of the ELM-free H-mode pedestal in NSTX. Journal of Nuclear Materials, 2013, 438, S979-S982.	2.7	9
110	Surface chemistry analysis of lithium conditioned NSTX graphite tiles correlated to plasma performance. Fusion Engineering and Design, 2013, 88, 3157-3164.	1.9	17
111	Study of non-axisymmetric divertor footprints using 2-D IR and visible cameras and a 3-D heat conduction solver in NSTX. Journal of Nuclear Materials, 2013, 438, S317-S320.	2.7	4
112	Characterization of fueling NSTX H-mode plasmas diverted to a liquid lithium divertor. Journal of Nuclear Materials, 2013, 438, S488-S492.	2.7	8
113	Effect of n=3 perturbation field amplitudes below the ELM triggering threshold on edge and SOL transport in NSTX. Journal of Nuclear Materials, 2013, 438, S388-S392.	2.7	3
114	Measurement and modeling of surface temperature dynamics of the NSTX liquid lithium divertor. Journal of Nuclear Materials, 2013, 438, S397-S400.	2.7	2
115	Results and future plans of the Lithium Tokamak eXperiment (LTX). Journal of Nuclear Materials, 2013, 438, S1096-S1099.	2.7	13
116	Liquid lithium divertor characteristics and plasmaâ€“material interactions in NSTX high-performance plasmas. Nuclear Fusion, 2013, 53, 083032.	3.5	79
117	The dependence of H-mode energy confinement and transport on collisionality in NSTX. Nuclear Fusion, 2013, 53, 063005.	3.5	40
118	Dependence of the Lâ€“H transition on X-point geometry and divertor recycling on NSTX. Nuclear Fusion, 2013, 53, 113032.	3.5	23
119	Scaling of the tokamak near the scrape-off layer H-mode power width and implications for ITER. Nuclear Fusion, 2013, 53, 093031.	3.5	448
120	Recent progress in the NSTX/NSTX-U lithium programme and prospects for reactor-relevant liquid-lithium based divertor development. Nuclear Fusion, 2013, 53, 113030.	3.5	32
121	Progress in characterization of the pedestal stability and turbulence during the edge-localized-mode cycle on National Spherical Torus Experiment. Nuclear Fusion, 2013, 53, 093026.	3.5	28
122	2D divertor heat flux distribution using a 3D heat conduction solver in National Spherical Torus Experiment. Review of Scientific Instruments, 2013, 84, 023505.	1.3	23
123	Power and particle exhaust in an ST-FNSF. , 2013, , .		2
124	First observations of ELM triggering by injected lithium granules in EAST. Nuclear Fusion, 2013, 53, 113023.	3.5	71
125	Experimental imaging of separatrix splitting on DIII-D. Nuclear Fusion, 2012, 52, 122001.	3.5	24
126	Spectral emission measurements of lithium on the lithium tokamak experiment. Review of Scientific Instruments, 2012, 83, 10D537.	1.3	3

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127	Overview of the physics and engineering design of NSTX upgrade. Nuclear Fusion, 2012, 52, 083015.	3.5	177
128	NSTX plasma operation with a Liquid Lithium Divertor. Fusion Engineering and Design, 2012, 87, 1724-1731.	1.9	72
129	Recent progress of NSTX lithium program and opportunities for magnetic fusion research. Fusion Engineering and Design, 2012, 87, 1770-1776.	1.9	11
130	The effect of progressively increasing lithium coatings on plasma discharge characteristics, transport, edge profiles and ELM stability in the National Spherical Torus Experiment. Nuclear Fusion, 2012, 52, 083001.	3.5	101
131	â€œSnowflakeâ€•divertor configuration in NSTX. Journal of Nuclear Materials, 2011, 415, S365-S368.	2.7	26
132	Deuterium retention in NSTX with lithium conditioning. Journal of Nuclear Materials, 2011, 415, S773-S776.	2.7	19
133	Turbulent transport and the scrape-off-layer width. Journal of Nuclear Materials, 2011, 415, S605-S608.	2.7	16
134	Measurements and 2-D modeling of recycling and edge transport in discharges with lithium-coated PFCs in NSTX. Journal of Nuclear Materials, 2011, 415, S409-S412.	2.7	41
135	NSTX plasma response to lithium coated divertor. Journal of Nuclear Materials, 2011, 415, S400-S404.	2.7	32
136	Dependence of divertor heat flux widths on heating power, flux expansion, and plasma current in the NSTX. Journal of Nuclear Materials, 2011, 415, S360-S364.	2.7	52
137	Characteristics of divertor heat and particle deposition with intrinsic and applied 3-D fields in NSTX H-mode plasmas. Journal of Nuclear Materials, 2011, 415, S918-S922.	2.7	6
138	Continuous Improvement of H-Mode Discharge Performance with Progressively Increasing Lithium Coatings in the National Spherical Torus Experiment. Physical Review Letters, 2011, 107, 145004.	7.8	77
139	The relationships between edge localized modes suppression, pedestal profiles and lithium wall coatings in NSTX. Plasma Physics and Controlled Fusion, 2011, 53, 105011.	2.1	43
140	Taming the plasmaâ€“material interface with the â€˜snowflakeâ€™ divertor in NSTX. Nuclear Fusion, 2011, 51, 012001.	3.5	73
141	Dynamical evolution of pedestal parameters in ELMy H-mode in the National Spherical Torus Experiment. Nuclear Fusion, 2011, 51, 103031.	3.5	31
142	Pedestal characterization and stability of small-ELM regimes in NSTX. Nuclear Fusion, 2011, 51, 103022.	3.5	17
143	Lâ€“H threshold studies in NSTX. Nuclear Fusion, 2011, 51, 113019.	3.5	29
144	Edge transport and turbulence reduction with lithium coated plasma facing components in the National Spherical Torus Experiment. Physics of Plasmas, 2011, 18, .	1.9	59

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145	Lithium coatings on NSTX plasma facing components and its effects on boundary control, core plasma performance, and operation. Fusion Engineering and Design, 2010, 85, 865-873.	1.9	35
146	Multi-Fluid Modeling of Low-Recycling Divertor Regimes. Contributions To Plasma Physics, 2010, 50, 299-305.	1.1	8
147	The impact of lithium wall coatings on NSTX discharges and the engineering of the Lithium Tokamak eXperiment (LTX). Fusion Engineering and Design, 2010, 85, 1283-1289.	1.9	17
148	Implications of NSTX lithium results for magnetic fusion research. Fusion Engineering and Design, 2010, 85, 882-889.	1.9	17
149	Experiments with liquid metal walls: Status of the lithium tokamak experiment. Fusion Engineering and Design, 2010, 85, 874-881.	1.9	28
150	First observation of ELM pacing with vertical jogs in a spherical torus. Nuclear Fusion, 2010, 50, 064015.	3.5	22
151	Progress in the development of ELM pace-making with non-axisymmetric magnetic perturbations in NSTX. Nuclear Fusion, 2010, 50, 064016.	3.5	14
152	2D soft x-ray system on DIII-D for imaging the magnetic topology in the pedestal region. Review of Scientific Instruments, 2010, 81, 10E534.	1.3	12
153	Simulation of a tangential soft x-ray imaging system. Review of Scientific Instruments, 2010, 81, 10E533.	1.3	4
154	On Demand Triggering of Edge Localized Instabilities Using External Nonaxisymmetric Magnetic Perturbations in Toroidal Plasmas. Physical Review Letters, 2010, 104, 045001.	7.8	66
155	Triggered Confinement Enhancement and Pedestal Expansion in High-Confinement-Mode Discharges in the National Spherical Torus Experiment. Physical Review Letters, 2010, 105, 135004.	7.8	41
156	Overview of Lâ€“H power threshold studies in NSTX. Nuclear Fusion, 2010, 50, 064010.	3.5	40
157	High speed infrared camera diagnostic for heat flux measurement in NSTX. Review of Scientific Instruments, 2010, 81, 023501.	1.3	38
158	Edge-Localized-Mode Suppression through Density-Profile Modification with Lithium-Wall Coatings in the National Spherical Torus Experiment. Physical Review Letters, 2009, 103, 075001.	7.8	154
159	Evaporated lithium surface coatings in NSTX. Journal of Nuclear Materials, 2009, 390-391, 1000-1004.	2.7	74
160	Physics design requirements for the National Spherical Torus Experiment liquid lithium divertor. Fusion Engineering and Design, 2009, 84, 1125-1129.	1.9	30
161	The enhanced pedestal H-mode in the National Spherical Torus experiment. Journal of Nuclear Materials, 2009, 390-391, 440-443.	2.7	6
162	Transition to ELM-free improved H-mode by lithium deposition on NSTX graphite divertor surfaces. Journal of Nuclear Materials, 2009, 390-391, 764-767.	2.7	67

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163	Overview of physics results from MAST. Nuclear Fusion, 2009, 49, 104017.	3.5	36
164	Pedestal stability comparison and ITER pedestal prediction. Nuclear Fusion, 2009, 49, 085035.	3.5	179
165	Overview of results from the National Spherical Torus Experiment (NSTX). Nuclear Fusion, 2009, 49, 104016.	3.5	41
166	Plasma response to lithium-coated plasma-facing components in the National Spherical Torus Experiment. Plasma Physics and Controlled Fusion, 2009, 51, 124054.	2.1	99
167	The effect of lithium surface coatings on plasma performance in the National Spherical Torus Experiment. Physics of Plasmas, 2008, 15, .	1.9	153
168	The role of parallel heat transport in the relation between upstream scrape-off layer widths and target heat flux width in H-mode plasmas of the National Spherical Torus Experiment. Physics of Plasmas, 2008, 15, .	1.9	5
169	Edge localized modes: recent experimental findings and related issues. Plasma Physics and Controlled Fusion, 2007, 49, S43-S62.	2.1	74
170	Confinement and local transport in the National Spherical Torus Experiment (NSTX). Nuclear Fusion, 2007, 47, 499-509.	3.5	98
171	A â€˜multi-colourâ€™ SXR diagnostic for time and space-resolved measurements of electron temperature, MHD activity and particle transport in MCF plasmas. Plasma Physics and Controlled Fusion, 2007, 49, 1245-1257.	2.1	21
172	Overview of physics results from MAST. Nuclear Fusion, 2007, 47, S658-S667.	3.5	25
173	Progress towards steady state at low aspect ratio on the National Spherical Torus Experiment (NSTX). Nuclear Fusion, 2007, 47, 1376-1382.	3.5	15
174	Overview of recent physics results from the National Spherical Torus Experiment (NSTX). Nuclear Fusion, 2007, 47, S645-S657.	3.5	40
175	Divertor heat flux reduction and detachment experiments in NSTX. Journal of Nuclear Materials, 2007, 363-365, 432-436.	2.7	17
176	Effect of lithium PFC coatings on NSTX density control. Journal of Nuclear Materials, 2007, 363-365, 791-796.	2.7	54
177	Resistive wall stabilized operation in rotating high beta NSTX plasmas. Nuclear Fusion, 2006, 46, 635-644.	3.5	137
178	New capabilities and results for the National Spherical Torus Experiment. Nuclear Fusion, 2006, 46, S565-S572.	3.5	28
179	Enhanced Energy Confinement and Performance in a Low-Recycling Tokamak. Physical Review Letters, 2006, 97, 075002.	7.8	133
180	Pedestal conditions for small ELM regimes in tokamaks. Plasma Physics and Controlled Fusion, 2006, 48, A171-A181.	2.1	88

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181	Progress towards steady state on NSTX. Nuclear Fusion, 2006, 46, S22-S28.	3.5	17
182	Modelling of stochastic magnetic perturbation by RWMEF coils on NSTX. Nuclear Fusion, 2006, 46, 858-863.	3.5	11
183	Effect of plasma shaping on performance in the National Spherical Torus Experiment. Physics of Plasmas, 2006, 13, 056122.	1.9	33
184	Characteristics and Contributions of the Three Major United States Toroidal Magnetic Fusion Facilities. Journal of Fusion Energy, 2005, 24, 125-171.	1.2	2
185	Requested Information Provided by the Three Major United States Toroidal Magnetic Fusion Facilities: Report of the 2005 FESAC Facilities Panel, Vol. 2. Journal of Fusion Energy, 2005, 24, 173-254.	1.2	0
186	Observation of a high performance operating regime with small edge-localized modes in the National Spherical Torus Experiment. Nuclear Fusion, 2005, 45, 264-270.	3.5	53
187	H-mode pedestal, ELM and power threshold studies in NSTX. Nuclear Fusion, 2005, 45, 1066-1077.	3.5	68
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