

Rajesh Maingi

List of Publications by Year in descending order

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274
papers

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280
docs citations

280
times ranked

2806
citing authors

#	ARTICLE	IF	CITATIONS
1	Scaling of the tokamak near the scrape-off layer H-mode power width and implications for ITER. Nuclear Fusion, 2013, 53, 093031.	3.5	448
2	Exploration of spherical torus physics in the NSTX device. Nuclear Fusion, 2000, 40, 557-561.	3.5	363
3	A long-pulse high-confinement plasma regime in the Experimental Advanced Superconducting Tokamak. Nature Physics, 2013, 9, 817-821.	16.7	234
4	Pedestal stability comparison and ITER pedestal prediction. Nuclear Fusion, 2009, 49, 085035.	3.5	179
5	Overview of the physics and engineering design of NSTX upgrade. Nuclear Fusion, 2012, 52, 083015.	3.5	177
6	Equilibrium properties of spherical torus plasmas in NSTX. Nuclear Fusion, 2001, 41, 1601-1611.	3.5	165
7	Edge-Localized-Mode Suppression through Density-Profile Modification with Lithium-Wall Coatings in the National Spherical Torus Experiment. Physical Review Letters, 2009, 103, 075001.	7.8	154
8	The effect of lithium surface coatings on plasma performance in the National Spherical Torus Experiment. Physics of Plasmas, 2008, 15, .	1.9	153
9	Resistive wall stabilized operation in rotating high beta NSTX plasmas. Nuclear Fusion, 2006, 46, 635-644.	3.5	137
10	Enhanced Energy Confinement and Performance in a Low-Recycling Tokamak. Physical Review Letters, 2006, 97, 075002.	7.8	133
11	Higher Fusion Power Gain with Current and Pressure Profile Control in Strongly Shaped DIII-D Tokamak Plasmas. Physical Review Letters, 1996, 77, 2714-2717.	7.8	128
12	Gyrokinetic projection of the divertor heat-flux width from present tokamaks to ITER. Nuclear Fusion, 2017, 57, 116023.	3.5	125
13	Enhanced H-mode pedestals with lithium injection in DIII-D. Nuclear Fusion, 2015, 55, 063018.	3.5	123
14	Fusion nuclear science facilities and pilot plants based on the spherical tokamak. Nuclear Fusion, 2016, 56, 106023.	3.5	119
15	The effect of plasma shape on H-mode pedestal characteristics on DIII-D. Plasma Physics and Controlled Fusion, 2000, 42, A175-A184.	2.1	114
16	Edge turbulence measurements in NSTX by gas puff imaging. Review of Scientific Instruments, 2001, 72, 931-934.	1.3	107
17	The effect of progressively increasing lithium coatings on plasma discharge characteristics, transport, edge profiles and ELM stability in the National Spherical Torus Experiment. Nuclear Fusion, 2012, 52, 083001.	3.5	101
18	Plasma response to lithium-coated plasma-facing components in the National Spherical Torus Experiment. Plasma Physics and Controlled Fusion, 2009, 51, 124054.	2.1	99

#	ARTICLE	IF	CITATIONS
19	Confinement and local transport in the National Spherical Torus Experiment (NSTX). Nuclear Fusion, 2007, 47, 499-509.	3.5	98
20	Recent liquid lithium limiter experiments in CDX-U. Nuclear Fusion, 2005, 45, 519-523.	3.5	95
21	New Steady-State Quiescent High-Confinement Plasma in an Experimental Advanced Superconducting Tokamak. Physical Review Letters, 2015, 114, 055001.	7.8	93
22	Internal kink mode dynamics in high- β^2 NSTX plasmas. Nuclear Fusion, 2005, 45, 539-556.	3.5	89
23	Pedestal conditions for small ELM regimes in tokamaks. Plasma Physics and Controlled Fusion, 2006, 48, A171-A181.	2.1	88
24	\hat{A} -Limiting MHD instabilities in improved-performance NSTX spherical torus plasmas. Nuclear Fusion, 2003, 43, 330-340.	3.5	83
25	Liquid lithium divertor characteristics and plasma-material interactions in NSTX high-performance plasmas. Nuclear Fusion, 2013, 53, 083032.	3.5	79
26	Radiative divertor experiments in DIII-D with D ₂ injection. Nuclear Fusion, 1997, 37, 321-338.	3.5	78
27	H-mode pedestal characteristics, ELMs, and energy confinement in ITER shape discharges on DIII-D. Plasma Physics and Controlled Fusion, 1998, 40, 845-850.	2.1	77
28	Continuous Improvement of H-Mode Discharge Performance with Progressively Increasing Lithium Coatings in the National Spherical Torus Experiment. Physical Review Letters, 2011, 107, 145004.	7.8	77
29	Helium transport and exhaust studies in enhanced confinement regimes in DIII-D. Physics of Plasmas, 1995, 2, 2357-2365.	1.9	76
30	The Filterscope. Review of Scientific Instruments, 2003, 74, 2068-2070.	1.3	74
31	Edge localized modes: recent experimental findings and related issues. Plasma Physics and Controlled Fusion, 2007, 49, S43-S62.	2.1	74
32	Evaporated lithium surface coatings in NSTX. Journal of Nuclear Materials, 2009, 390-391, 1000-1004.	2.7	74
33	Taming the plasma-material interface with the "snowflake" divertor in NSTX. Nuclear Fusion, 2011, 51, 012001.	3.5	73
34	NSTX plasma operation with a Liquid Lithium Divertor. Fusion Engineering and Design, 2012, 87, 1724-1731.	1.9	72
35	First observations of ELM triggering by injected lithium granules in EAST. Nuclear Fusion, 2013, 53, 113023.	3.5	71
36	Physics of pedestal density profile formation and its impact on H-mode density limit in burning plasmas. Physics of Plasmas, 2003, 10, 3984-3991.	1.9	69

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37	Slow \tilde{H} Transitions in DIII-D Plasmas. <i>Physical Review Letters</i> , 2002, 88, 255002.	7.8	68
38	H-mode pedestal, ELM and power threshold studies in NSTX. <i>Nuclear Fusion</i> , 2005, 45, 1066-1077.	3.5	68
39	Integrated operation of steady-state long-pulse H-mode in Experimental Advanced Superconducting Tokamak. <i>Nuclear Fusion</i> , 2019, 59, 086030.	3.5	68
40	Transition to ELM-free improved H-mode by lithium deposition on NSTX graphite divertor surfaces. <i>Journal of Nuclear Materials</i> , 2009, 390-391, 764-767.	2.7	67
41	Non-inductive current generation in NSTX using coaxial helicity injection. <i>Nuclear Fusion</i> , 2001, 41, 1081-1086.	3.5	66
42	On Demand Triggering of Edge Localized Instabilities Using External Nonaxisymmetric Magnetic Perturbations in Toroidal Plasmas. <i>Physical Review Letters</i> , 2010, 104, 045001.	7.8	66
43	Survey of target plate heat flux in diverted DIII-D tokamak discharges. <i>Nuclear Fusion</i> , 1998, 38, 1225-1249.	3.5	65
44	Beta-limiting instabilities and global mode stabilization in the National Spherical Torus Experiment. <i>Physics of Plasmas</i> , 2002, 9, 2085-2092.	1.9	65
45	Distributed Divertor Radiation through Convection in DIII-D. <i>Physical Review Letters</i> , 1997, 78, 4769-4772.	7.8	63
46	Electric field-induced plasma convection in tokamak divertors. <i>Physics of Plasmas</i> , 2000, 7, 1075-1078.	1.9	61
47	The two-dimensional structure of radiative divertor plasmas in the DIII-D tokamak. <i>Physics of Plasmas</i> , 1997, 4, 1761-1773.	1.9	60
48	Progress towards high performance plasmas in the National Spherical Torus Experiment (NSTX). <i>Nuclear Fusion</i> , 2005, 45, S168-S180.	3.5	60
49	Edge transport and turbulence reduction with lithium coated plasma facing components in the National Spherical Torus Experiment. <i>Physics of Plasmas</i> , 2011, 18, .	1.9	59
50	High performance H mode plasmas at densities above the Greenwald limit. <i>Nuclear Fusion</i> , 2002, 42, 52-58.	3.5	57
51	First results of the use of a continuously flowing lithium limiter in high performance discharges in the EAST device. <i>Nuclear Fusion</i> , 2016, 56, 046011.	3.5	57
52	Effect of lithium PFC coatings on NSTX density control. <i>Journal of Nuclear Materials</i> , 2007, 363-365, 791-796.	2.7	54
53	Approaches towards long-pulse divertor operations on EAST by active control of plasma-wall interactions. <i>Nuclear Fusion</i> , 2014, 54, 013002.	3.5	54
54	Investigation of physical processes limiting plasma density in high confinement mode discharges on DIII-D. <i>Physics of Plasmas</i> , 1997, 4, 1752-1760.	1.9	53

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55	Impurity enrichment studies with induced scrape-off layer flow on DIII-D. Nuclear Fusion, 1998, 38, 1839-1859.	3.5	53
56	The resistive wall mode and feedback control physics design in NSTX. Nuclear Fusion, 2004, 44, 560-570.	3.5	53
57	Observation of a high performance operating regime with small edge-localized modes in the National Spherical Torus Experiment. Nuclear Fusion, 2005, 45, 264-270.	3.5	53
58	Overview of physics results from the conclusive operation of the National Spherical Torus Experiment. Nuclear Fusion, 2013, 53, 104007.	3.5	53
59	Dependence of divertor heat flux widths on heating power, flux expansion, and plasma current in the NSTX. Journal of Nuclear Materials, 2011, 415, S360-S364.	2.7	52
60	Edge microstability of NSTX plasmas without and with lithium-coated plasma-facing components. Nuclear Fusion, 2013, 53, 113016.	3.5	52
61	Physics of the detached radiative divertor regime in DIII-D. Plasma Physics and Controlled Fusion, 1999, 41, A345-A355.	2.1	49
62	Overview of the initial NSTX experimental results. Nuclear Fusion, 2001, 41, 1435-1447.	3.5	49
63	The national spherical torus experiment (NSTX) research programme and progress towards high beta, long pulse operating scenarios. Nuclear Fusion, 2003, 43, 1653-1664.	3.5	49
64	Effect of edge neutrals on the low-to-high confinement transition threshold in the DIII-D tokamak. Physics of Plasmas, 1998, 5, 2623-2636.	1.9	47
65	Aspect ratio scaling of ideal no-wall stability limits in high bootstrap fraction tokamak plasmas. Physics of Plasmas, 2004, 11, 639-646.	1.9	47
66	Initial physics results from the National Spherical Torus Experiment. Physics of Plasmas, 2001, 8, 1977-1987.	1.9	46
67	Overview of NSTX Upgrade initial results and modelling highlights. Nuclear Fusion, 2017, 57, 102006.	3.5	45
68	ELM elimination with Li powder injection in EAST discharges using the tungsten upper divertor. Nuclear Fusion, 2018, 58, 024003.	3.5	45
69	Impurity reduction during 'puff and pump' experiments on DIII-D. Nuclear Fusion, 1995, 35, 1000-1007.	3.5	44
70	Control of wall particle inventory with divertor pumping on DIII-D. Nuclear Fusion, 1996, 36, 245-253.	3.5	44
71	H-mode research in NSTX. Nuclear Fusion, 2003, 43, 969-974.	3.5	44
72	The relationships between edge localized modes suppression, pedestal profiles and lithium wall coatings in NSTX. Plasma Physics and Controlled Fusion, 2011, 53, 105011.	2.1	43

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73	Promising High-Confinement Regime for Steady-State Fusion. <i>Physical Review Letters</i> , 2019, 122, 255001.	7.8	43
74	High frequency pacing of edge localized modes by injection of lithium granules in DIII-D H-mode discharges. <i>Nuclear Fusion</i> , 2016, 56, 056008.	3.5	42
75	Overview of results from the National Spherical Torus Experiment (NSTX). <i>Nuclear Fusion</i> , 2009, 49, 104016.	3.5	41
76	Triggered Confinement Enhancement and Pedestal Expansion in High-Confinement-Mode Discharges in the National Spherical Torus Experiment. <i>Physical Review Letters</i> , 2010, 105, 135004.	7.8	41
77	Measurements and 2-D modeling of recycling and edge transport in discharges with lithium-coated PFCs in NSTX. <i>Journal of Nuclear Materials</i> , 2011, 415, S409-S412.	2.7	41
78	Recent DIII-D divertor research. <i>Plasma Physics and Controlled Fusion</i> , 1995, 37, A191-A202.	2.1	40
79	Overview of recent physics results from the National Spherical Torus Experiment (NSTX). <i>Nuclear Fusion</i> , 2007, 47, S645-S657.	3.5	40
80	Overview of H power threshold studies in NSTX. <i>Nuclear Fusion</i> , 2010, 50, 064010.	3.5	40
81	The dependence of H-mode energy confinement and transport on collisionality in NSTX. <i>Nuclear Fusion</i> , 2013, 53, 063005.	3.5	40
82	A multi-species powder dropper for magnetic fusion applications. <i>Review of Scientific Instruments</i> , 2018, 89, 10K121.	1.3	40
83	Measurement of neutral density near the X point in the DIII-D tokamak. <i>Nuclear Fusion</i> , 2000, 40, 175-180.	3.5	39
84	Enhanced confinement scenarios without large edge localized modes in tokamaks: control, performance, and extrapolability issues for ITER. <i>Nuclear Fusion</i> , 2014, 54, 114016.	3.5	39
85	Confinement studies of auxiliary heated NSTX plasmas. <i>Nuclear Fusion</i> , 2004, 44, 513-523.	3.5	38
86	High speed infrared camera diagnostic for heat flux measurement in NSTX. <i>Review of Scientific Instruments</i> , 2010, 81, 023501.	1.3	38
87	Overview of physics studies on ASDEX Upgrade. <i>Nuclear Fusion</i> , 2019, 59, 112014.	3.5	38
88	Testing of liquid lithium limiters in CDX-U. <i>Fusion Engineering and Design</i> , 2004, 72, 121-132.	1.9	37
89	Overview of physics results from MAST. <i>Nuclear Fusion</i> , 2009, 49, 104017.	3.5	36
90	Impact of wall materials and seeding gases on the pedestal and on core plasma performance. <i>Nuclear Materials and Energy</i> , 2017, 12, 18-27.	1.3	36

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91	Effect of boronization on ohmic plasmas in NSTX. Nuclear Fusion, 2002, 42, 329-332.	3.5	35
92	Lithium coatings on NSTX plasma facing components and its effects on boundary control, core plasma performance, and operation. Fusion Engineering and Design, 2010, 85, 865-873.	1.9	35
93	Real-time wall conditioning by controlled injection of boron and boron nitride powder in full tungsten wall ASDEX Upgrade. Nuclear Materials and Energy, 2019, 19, 384-389.	1.3	35
94	Helium Exhaust Studies in H-Mode Discharges in the DIII-D Tokamak Using an Argon-Frosted Divertor Cryopump. Physical Review Letters, 1995, 74, 2702-2705.	7.8	34
95	Pump plenum pressure dependence on divertor plasma parameters and magnetic geometry in the DIII-D tokamak. Nuclear Fusion, 1999, 39, 1187-1192.	3.5	34
96	Characteristics of the First H-Mode Discharges in the National Spherical Torus Experiment. Physical Review Letters, 2002, 88, 035003.	7.8	34
97	Controlled Optimization of Mode Conversion from Electron Bernstein Waves to Extraordinary Mode in Magnetized Plasma. Physical Review Letters, 2003, 90, 165001.	7.8	34
98	Effect of gas fuelling location on H-mode access in NSTX. Plasma Physics and Controlled Fusion, 2004, 46, A305-A313.	2.1	33
99	Effect of plasma shaping on performance in the National Spherical Torus Experiment. Physics of Plasmas, 2006, 13, 056122.	1.9	33
100	Results from an improved flowing liquid lithium limiter with increased flow uniformity in high power plasmas in EAST. Nuclear Fusion, 2019, 59, 016009.	3.5	33
101	Suppression of edge localized modes with real-time boron injection using the tungsten divertor in EAST. Nuclear Fusion, 2021, 61, 014002.	3.5	33
102	Measurement of electron particle transport coefficients in different operational modes of DIII-D. Nuclear Fusion, 1998, 38, 485-494.	3.5	32
103	Plasma-lithium interaction in the CDX-U spherical torus. Fusion Engineering and Design, 2002, 60, 157-166.	1.9	32
104	NSTX plasma response to lithium coated divertor. Journal of Nuclear Materials, 2011, 415, S400-S404.	2.7	32
105	Recent progress in the NSTX/NSTX-U lithium programme and prospects for reactor-relevant liquid-lithium based divertor development. Nuclear Fusion, 2013, 53, 113030.	3.5	32
106	Dynamical evolution of pedestal parameters in ELMy H-mode in the National Spherical Torus Experiment. Nuclear Fusion, 2011, 51, 103031.	3.5	31
107	of Plasmas, 2015, 22, 056112.	1.9	31
108	Active conditioning of ASDEX Upgrade tungsten plasma-facing components and discharge enhancement through boron and boron nitride particulate injection. Nuclear Fusion, 2019, 59, 126034.	3.5	31

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109	Physics design requirements for the National Spherical Torus Experiment liquid lithium divertor. Fusion Engineering and Design, 2009, 84, 1125-1129.	1.9	30
110	Heat flux management via advanced magnetic divertor configurations and divertor detachment. Journal of Nuclear Materials, 2015, 463, 1186-1190.	2.7	30
111	Radiative divertor and scrape-off layer experiments in open and baffled divertors on DIII-D. Nuclear Fusion, 1999, 39, 2015-2023.	3.5	29
112	Physics of slow L-H transitions in the DIII-D tokamak. Nuclear Fusion, 2002, 42, 1134-1143.	3.5	29
113	Lâ€H threshold studies in NSTX. Nuclear Fusion, 2011, 51, 113019.	3.5	29
114	Partially detached radiative divertor with active divertor pumping. Nuclear Fusion, 1997, 37, 643-655.	3.5	28
115	Radiative divertor plasmas with convection in DIII-D. Physics of Plasmas, 1998, 5, 1736-1743.	1.9	28
116	New capabilities and results for the National Spherical Torus Experiment. Nuclear Fusion, 2006, 46, S565-S572.	3.5	28
117	Experiments with liquid metal walls: Status of the lithium tokamak experiment. Fusion Engineering and Design, 2010, 85, 874-881.	1.9	28
118	Progress in characterization of the pedestal stability and turbulence during the edge-localized-mode cycle on National Spherical Torus Experiment. Nuclear Fusion, 2013, 53, 093026.	3.5	28
119	Compatibility of lithium plasma-facing surfaces with high edge temperatures in the Lithium Tokamak Experiment. Physics of Plasmas, 2017, 24, .	1.9	28
120	H-mode threshold and dynamics in the National Spherical Torus Experiment. Physics of Plasmas, 2003, 10, 1755-1764.	1.9	27
121	Formation of transport barriers in the MAST spherical tokamak. Plasma Physics and Controlled Fusion, 2004, 46, A291-A298.	2.1	27
122	Observations of wall conditioning by means of boron powder injection in DIII-D H-mode plasmas. Nuclear Fusion, 2020, 60, 126010.	3.5	27
123	â€Snowflakeâ€•divertor configuration in NSTX. Journal of Nuclear Materials, 2011, 415, S365-S368.	2.7	26
124	Progress towards high-performance, steady-state spherical torus. Plasma Physics and Controlled Fusion, 2003, 45, A335-A350.	2.1	25
125	Overview of physics results from MAST. Nuclear Fusion, 2007, 47, S658-S667.	3.5	25
126	Recent advances in long-pulse high-confinement plasma operations in Experimental Advanced Superconducting Tokamak. Physics of Plasmas, 2014, 21, 056107.	1.9	25

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127	Real time wall conditioning with lithium powder injection in long pulse H-mode plasmas in EAST with tungsten divertor. Nuclear Materials and Energy, 2019, 19, 124-130.	1.3	25
128	Ohmic flux consumption during initial operation of the NSTX spherical torus. Nuclear Fusion, 2001, 41, 1197-1206.	3.5	24
129	Experimental imaging of separatrix splitting on DIII-D. Nuclear Fusion, 2012, 52, 122001.	3.5	24
130	Advanced divertor configurations with large flux expansion. Journal of Nuclear Materials, 2013, 438, S96-S101.	2.7	24
131	Assessment of effects of neutrals on the power threshold for L-H transitions in DIII-D. Plasma Physics and Controlled Fusion, 1998, 40, 717-720.	2.1	23
132	Initial studies of core and edge transport of NSTX plasmas. Plasma Physics and Controlled Fusion, 2002, 44, A165-A173.	2.1	23
133	Recent results from the National Spherical Torus Experiment. Plasma Physics and Controlled Fusion, 2003, 45, 657-669.	2.1	23
134	Dependence of the L-H transition on X-point geometry and divertor recycling on NSTX. Nuclear Fusion, 2013, 53, 113032.	3.5	23
135	2D divertor heat flux distribution using a 3D heat conduction solver in National Spherical Torus Experiment. Review of Scientific Instruments, 2013, 84, 023505.	1.3	23
136	Divertor neutral pressure enhancement with a baffle in DIII-D. Nuclear Fusion, 1993, 33, 533-545.	3.5	22
137	Enhanced conversion of thermal electron Bernstein waves to the extraordinary electromagnetic mode on the National Spherical Torus Experiment. Physics of Plasmas, 2003, 10, 1395-1401.	1.9	22
138	First observation of ELM pacing with vertical jogs in a spherical torus. Nuclear Fusion, 2010, 50, 064015.	3.5	22
139	A "multi-colour"™ SXR diagnostic for time and space-resolved measurements of electron temperature, MHD activity and particle transport in MCF plasmas. Plasma Physics and Controlled Fusion, 2007, 49, 1245-1257.	2.1	21
140	An overview of recent physics results from NSTX. Nuclear Fusion, 2015, 55, 104002.	3.5	21
141	Application of a species-selective Penning gauge to the measurement of neon and hydrogen-isotope partial pressures in the plasma boundary. Review of Scientific Instruments, 1997, 68, 400-403.	1.3	20
142	Effect of heating scheme on SOL width in DIII-D and EAST. Nuclear Materials and Energy, 2017, 12, 221-226.	1.3	20
143	Injected mass deposition thresholds for lithium granule instigated triggering of edge localized modes on EAST. Nuclear Fusion, 2018, 58, 036007.	3.5	20
144	NSTX/NSTX-U theory, modeling and analysis results. Nuclear Fusion, 2019, 59, 112007.	3.5	20

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145	Multi-faceted asymmetric radiation from the edge formation in DIII-D high-confinement mode discharges with continuous gas puffing. <i>Physics of Plasmas</i> , 1999, 6, 3941-3954.	1.9	19
146	Impact of the wall conditioning program on plasma performance in NSTX. <i>Journal of Nuclear Materials</i> , 2003, 313-316, 187-193.	2.7	19
147	Deuterium retention in NSTX with lithium conditioning. <i>Journal of Nuclear Materials</i> , 2011, 415, S773-S776.	2.7	19
148	Liquid lithium loop system to solve challenging technology issues for fusion power plant. <i>Nuclear Fusion</i> , 2017, 57, 116056.	3.5	19
149	Experiments of continuously and stably flowing lithium limiter in EAST towards a solution for the power exhaust of future fusion devices. <i>Nuclear Materials and Energy</i> , 2019, 18, 99-104.	1.3	19
150	High-resolution spectroscopic diagnostic for divertor and scrape-off layer neutral and impurity emission measurements in the National Spherical Torus Experiment. <i>Review of Scientific Instruments</i> , 2003, 74, 2094-2097.	1.3	18
151	Infrared camera diagnostic for heat flux measurements on the National Spherical Torus Experiment. <i>Review of Scientific Instruments</i> , 2003, 74, 5090-5092.	1.3	18
152	Application of the radiating divertor approach to innovative tokamak divertor concepts. <i>Journal of Nuclear Materials</i> , 2015, 463, 1225-1228.	2.7	18
153	Study of the phase transition dynamics of the L to H transition. <i>Plasma Physics and Controlled Fusion</i> , 1999, 41, 243-249.	2.1	17
154	Progress towards steady state on NSTX. <i>Nuclear Fusion</i> , 2006, 46, S22-S28.	3.5	17
155	Divertor heat flux reduction and detachment experiments in NSTX. <i>Journal of Nuclear Materials</i> , 2007, 363-365, 432-436.	2.7	17
156	The impact of lithium wall coatings on NSTX discharges and the engineering of the Lithium Tokamak eXperiment (LTX). <i>Fusion Engineering and Design</i> , 2010, 85, 1283-1289.	1.9	17
157	Implications of NSTX lithium results for magnetic fusion research. <i>Fusion Engineering and Design</i> , 2010, 85, 882-889.	1.9	17
158	Pedestal characterization and stability of small-ELM regimes in NSTX. <i>Nuclear Fusion</i> , 2011, 51, 103022.	3.5	17
159	Surface chemistry analysis of lithium conditioned NSTX graphite tiles correlated to plasma performance. <i>Fusion Engineering and Design</i> , 2013, 88, 3157-3164.	1.9	17
160	Dependence of recycling and edge profiles on lithium evaporation in high triangularity, high performance NSTX H-mode discharges. <i>Journal of Nuclear Materials</i> , 2015, 463, 1134-1137.	2.7	17
161	Overview of impurity control and wall conditioning in NSTX. <i>Journal of Nuclear Materials</i> , 2001, 290-293, 1185-1189.	2.7	16
162	Initial results from coaxial helicity injection experiments in NSTX. <i>Plasma Physics and Controlled Fusion</i> , 2001, 43, 305-312.	2.1	16

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163	Turbulent transport and the scrape-off-layer width. <i>Journal of Nuclear Materials</i> , 2011, 415, S605-S608.	2.7	16
164	Progress toward commissioning and plasma operation in NSTX-U. <i>Nuclear Fusion</i> , 2015, 55, 073007.	3.5	16
165	Conference Report on the 4rd International Symposium on Lithium Applications. <i>Nuclear Fusion</i> , 2016, 56, 127002.	3.5	16
166	Upgraded flowing liquid lithium limiter for improving Li coverage uniformity and erosion resistance in EAST device. <i>Review of Scientific Instruments</i> , 2017, 88, 123506.	1.3	16
167	Real-time reduction of tungsten impurity influx using lithium powder injection in EAST. <i>Fusion Engineering and Design</i> , 2018, 137, 202-208.	1.9	16
168	Comparison of a simple recycling model and a complex recycling model in edge plasma transport calculations. <i>Nuclear Fusion</i> , 1994, 34, 283-287.	3.5	15
169	Enhancement of mode-converted electron Bernstein wave emission during National Spherical Torus Experiment H-mode plasmas. <i>Physics of Plasmas</i> , 2002, 9, 167-170.	1.9	15
170	Progress towards steady state at low aspect ratio on the National Spherical Torus Experiment (NSTX). <i>Nuclear Fusion</i> , 2007, 47, 1376-1382.	3.5	15
171	The effects of increasing lithium deposition on the power exhaust channel in NSTX. <i>Nuclear Fusion</i> , 2014, 54, 023001.	3.5	15
172	Mitigation of divertor heat flux by high-frequency ELM pacing with non-fuel pellet injection in DIII-D. <i>Nuclear Materials and Energy</i> , 2017, 12, 1030-1036.	1.3	15
173	Progress in the development of ELM pace-making with non-axisymmetric magnetic perturbations in NSTX. <i>Nuclear Fusion</i> , 2010, 50, 064016.	3.5	14
174	Snowflake Divertor Experiments in the DIII-D, NSTX, and NSTX-U Tokamaks Aimed at the Development of the Divertor Power Exhaust Solution. <i>IEEE Transactions on Plasma Science</i> , 2016, 44, 3445-3455.	1.3	14
175	Effect of lithium coating on long pulse high performance plasma discharges in EAST. <i>Plasma Physics and Controlled Fusion</i> , 2020, 62, 085012.	2.1	14
176	ELM Suppression by Boron Powder Injection and Comparison with Lithium Powder Injection on EAST. <i>Journal of Fusion Energy</i> , 2020, 39, 429-435.	1.2	14
177	EAST steady-state long pulse H-mode with core-edge integration for CFETR. <i>Nuclear Fusion</i> , 2022, 62, 076009.	3.5	14
178	Corrosion characteristics of Mo and TZM alloy for plasma facing components in molten lithium at 623ÅK. <i>Corrosion Science</i> , 2022, 200, 110202.	6.6	14
179	Results and future plans of the Lithium Tokamak eXperiment (LTX). <i>Journal of Nuclear Materials</i> , 2013, 438, S1096-S1099.	2.7	13
180	Latest Results From the Hybrid Illinois Device for Research and Applications (HIDRA). <i>IEEE Transactions on Plasma Science</i> , 2018, 46, 2685-2690.	1.3	13

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