Karl Krieger

List of Publications by Year
in descending order

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200 papers	7,915 citations	61984 43 h-index	60623 81 g-index
200 all docs	200 docs citations	200 times ranked	3103 citing authors

#	Article	IF	Citations
1	Chapter 4: Power and particle control. Nuclear Fusion, 2007, 47, S203-S263.	3.5	891
2	Recent analysis of key plasma wall interactions issues for ITER. Journal of Nuclear Materials, 2009, 390-391, 1-9.	2.7	671
3	Key ITER plasma edge and plasmaâ€“material interaction issues. Journal of Nuclear Materials, 2003, 313-316, 11-22.	2.7	319
4	Fuel retention studies with the ITER-Like Wall in JET. Nuclear Fusion, 2013, 53, 083023.	3.5	193
5	Plasmaâ€“surface interaction, scrape-off layer and divertor physics: implications for ITER. Nuclear Fusion, 2007, 47, 1189-1205.	3.5	156
6	Overview on plasma operation with a full tungsten wall in ASDEX Upgrade. Journal of Nuclear Materials, 2013, 438, S34-S41.	2.7	156
7	Improved Confinement in High-Density Ohmic Discharges in ASDEX. Physical Review Letters, 1988, 61, 1105-1108.	7.8	153
8	Observation of Continuous Divertor Detachment inH-Mode Discharges in ASDEX Upgrade. Physical Review Letters, 1995, 74, 4217-4220.	7.8	152
9	Overview of the JET results in support to ITER. Nuclear Fusion, 2017, 57, 102001.	3.5	150
10	Physics conclusions in support of ITER W divertor monoblock shaping. Nuclear Materials and Energy, 2017, 12, 60-74.	1.3	128
11	Conclusions about the use of tungsten in the divertor of ASDEX Upgrade. Journal of Nuclear Materials, 1999, 266-269, 207-216.	2.7	126
12	Tungsten divertor erosion in all metal devices: Lessons from the ITER like wall of JET. Journal of Nuclear Materials, 2013, 438, S42-S47.	2.7	116
13	Impurity behaviour in the ASDEX Upgrade divertor tokamak with large area tungsten walls. Plasma Physics and Controlled Fusion, 2002, 44, 811-826.	2.1	114
14	Plasma wall interaction and its implication in an all tungsten divertor tokamak. Plasma Physics and Controlled Fusion, 2007, 49, B59-B70.	2.1	110
15	Steady state advanced scenarios at ASDEX Upgrade. Plasma Physics and Controlled Fusion, 2002, 44, B69-B83.	2.1	108
16	Material erosion and migration in tokamaks. Plasma Physics and Controlled Fusion, 2005, 47, B303-B322.	2.1	105
17	Overview of the JET preparation for deuteriumâ€“tritium operation with the ITER like-wall. Nuclear Fusion, 2019, 59, 112021.	3.5	87
18	Tritium retention in next step devices and the requirements for mitigation and removal techniques. Plasma Physics and Controlled Fusion, 2006, 48, B189-B199.	2.1	83

#	Article	IF	Citations
19	Beryllium migration in JET ITER-like wall plasmas. Nuclear Fusion, 2015, 55, 063021.	3.5	83
20	ELM-induced transient tungsten melting in the JET divertor. Nuclear Fusion, 2015, 55, 023010.	3.5	83
21	The tungsten divertor experiment at ASDEX Upgrade. Plasma Physics and Controlled Fusion, 1996, 38, A165-A179.	2.1	82
22	The compatibility of high confinement times and complete divertor detachment in ASDEX-Upgrade. Plasma Physics and Controlled Fusion, 1995, 37, A37-A51.	2.1	80
23	Final steps to an all tungsten divertor tokamak. Journal of Nuclear Materials, 2007, 363-365, 52-59.	2.7	80
24	Determination of particle transport coefficients in ASDEX by gas modulation. Nuclear Fusion, 1992, 32, 217-237.	3.5	75
25	Spectroscopic measurements of tungsten erosion in the ASDEX Upgrade divertor. Plasma Physics and Controlled Fusion, 1997, 39, 1487-1499.	2.1	75
26	Residual carbon content in the initial ITER-Like Wall experiments at JET. Journal of Nuclear Materials, 2013, 438, S303-S308.	2.7	75
27	New results from the tungsten programme at ASDEX Upgrade. Journal of Nuclear Materials, 2003, 313-316, 116-126.	2.7	73
28	WALLDYN simulations of global impurity migration in JET and extrapolations to ITER. Nuclear Fusion, 2015, 55, 053015.	3.5	67
29	Integration of a radiative divertor for heat load control into JET high triangularity ELMy H-mode plasmas. Nuclear Fusion, 2012, 52, 063022.	3.5	58
30	Tungsten as plasma-facing material in ASDEX Upgrade. Fusion Engineering and Design, 2003, 65, 367-374.	1.9	57
31	Study of physical and chemical assisted physical sputtering of beryllium in the JET ITER-like wall. Nuclear Fusion, 2014, 54, 103001.	3.5	55
32	Impurity transport and neoclassical predictions. Plasma Physics and Controlled Fusion, 1991, 33, 1677-1695.	2.1	54
33	ELM induced tungsten melting and its impact on tokamak operation. Journal of Nuclear Materials, 2015, 463, 78-84.	2.7	53
34	An integrated model of impurity migration and wall composition dynamics for tokamaks. Journal of Nuclear Materials, 2011, 415, S284-S288.	2.7	52
35	Recrystallization and grain growth behavior of rolled tungsten under VDE-like short pulse high heat flux loads. Journal of Nuclear Materials, 2013, 433, 523-530.	2.7	50
36	Erosion and redeposition in the ASDEX Upgrade divertor. Journal of Nuclear Materials, 1994, 210, 43-50.	2.7	49

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37	Quantitative modeling of fuel retention in the JET-C and JET-ILW wall configurations by WallDYN and predictions for ITER. Journal of Nuclear Materials, 2015, 463, 66-72.	2.7	49
38	Overview of ASDEX Upgrade results. Nuclear Fusion, 1999, 39, 1321-1336.	3.5	47
39	Effect of surface roughness and substrate material on carbon erosion and deposition in the TEXTOR tokamak. Plasma Physics and Controlled Fusion, 2008, 50, 095008.	2.1	47
40	Tungsten erosion and redeposition in the all-tungsten divertor of ASDEX Upgrade. Physica Scripta, 2009, T138, 014039.	2.5	47
41	Carbon layers in the divertor of ASDEX Upgrade. Journal of Nuclear Materials, 2001, 290-293, 317-320.	2.7	46
42	Formation of deuteriumâ€“carbon inventories in gaps of plasma facing components. Journal of Nuclear Materials, 2007, 363-365, 870-876.	2.7	45
43	Operational conditions in a W-clad tokamak. Journal of Nuclear Materials, 2007, 367-370, 1497-1502.	2.7	45
44	Arcing in ASDEX Upgrade with a tungsten first wall. Journal of Nuclear Materials, 2009, 390-391, 747-750.	2.7	43
45	Erosion and deposition in the ASDEX Upgrade tungsten divertor experiment. Journal of Nuclear Materials, 1999, 266-269, 1003-1008.	2.7	40
46	Determination of the tungsten divertor retention at ASDEX Upgrade using a sublimation probe. Plasma Physics and Controlled Fusion, 2002, 44, 2091-2100.	2.1	40
47	Operating a full tungsten actively cooled tokamak: overview of WEST first phase of operation. Nuclear Fusion, 2022, 62, 042007.	3.5	39
48	Experiments on transient melting of tungsten by ELMs in ASDEX Upgrade. Nuclear Fusion, 2018, 58, 026024.	3.5	38
49	Overview of physics studies on ASDEX Upgrade. Nuclear Fusion, 2019, 59, 112014.	3.5	38
50	Wall conditioning in ASDEX Upgrade. Journal of Nuclear Materials, 2007, 363-365, 1369-1374.	2.7	36
51	Overview of ASDEX Upgrade results. Nuclear Fusion, 2013, 53, 104003.	3.5	36
52	Molecular impurities in ASDEX UPGRADE plasma discharges. Journal of Nuclear Materials, 1995, 220-222, 36-49.	2.7	35
53	Carbon erosion and deposition on the ASDEX Upgrade divertor tiles. Journal of Nuclear Materials, 2005, 337-339, 119-123.	2.7	35
54	Long-term evolution of the impurity composition and impurity events with the ITER-like wall at JET. Nuclear Fusion, 2013, 53, 073043.	3.5	35

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55	Solid tungsten Divertor-III for ASDEX Upgrade and contributions to ITER. Nuclear Fusion, 2015, 55, 063015.	3.5	35
56	Plasma operation with tungsten tiles at the central column of ASDEX Upgrade. Journal of Nuclear Materials, 2001, 290-293, 206-210.	2.7	34
57	Overview of ASDEX Upgrade results. Nuclear Fusion, 2001, 41, 1369-1389.	3.5	34
58	Induced tungsten melting events in the divertor of ASDEX Upgrade and their influence on plasma performance. Journal of Nuclear Materials, 2011, 415, S297-S300.	2.7	34
59	Blistering and re-deposition on tungsten exposed to ASDEX Upgrade divertor plasma. Journal of Nuclear Materials, 2013, 438, S220-S223.	2.7	34
60	Filament transport, warm ions and erosion in ASDEX Upgrade L-modes. Nuclear Fusion, 2015, 55, 033018.	3.5	34
61	Dependence on plasma shape and plasma fueling for small edge-localized mode regimes in TCV and ASDEX Upgrade. Nuclear Fusion, 2019, 59, 086020.	3.5	34
62	Recent results from divertor operation in ASDEX Upgrade. Plasma Physics and Controlled Fusion, 1994, 36, B79-B92.	2.1	33
63	Investigations of castellated structures for ITER: The effect of castellation shaping and alignment on fuel retention and impurity deposition in gaps. Journal of Nuclear Materials, 2009, 390-391, 556-559.	2.7	32
64	DIVIMP modeling of tungsten impurity transport in ITER. Journal of Nuclear Materials, 2007, 363-365, 674-679.	2.7	31
65	Evolution of surface melt damage, its influence on plasma performance and prospects of recovery. Journal of Nuclear Materials, 2013, 438, S27-S33.	2.7	31
66	A large divertor manipulator for ASDEX Upgrade. Fusion Engineering and Design, 2015, 98-99, 1496-1499.	1.9	31
67	Beryllium migration and evolution of first wall surface composition in the JET ILW configuration. Journal of Nuclear Materials, 2013, 438, S262-S266.	2.7	30
68	Resolidification-controlled melt dynamics under fast transient tokamak plasma loads. Nuclear Fusion, 2020, 60, 104001.	3.5	30
69	Performance of tungsten coatings as plasma facing components used in ASDEX Upgrade. Journal of Nuclear Materials, 1998, 258-263, 921-926.	2.7	29
70	Consequences of deuterium retention and release from Be-containing mixed materials for ITER Tritium Inventory Control. Journal of Nuclear Materials, 2011, 415, S731-S734.	2.7	29
71	Overview of experimental preparation for the ITER-Like Wall at JET. Journal of Nuclear Materials, 2011, 415, S936-S942.	2.7	29
72	Overview of ASDEX Upgrade resultsâ€"development of integrated operating scenarios for ITER. Nuclear Fusion, 2005, 45, S98-S108.	3.5	28

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73	EMC3-Eirene simulations of the spatial dependence of the tungsten divertor retention in ASDEX Upgrade. Plasma Physics and Controlled Fusion, 2011, 53, 125010.	2.1	28
74	Auxiliary heated multipellet-fuelled discharges in ASDEX and influence of density profile shape on confinement. Plasma Physics and Controlled Fusion, 1988, 30, 1611-1623.	2.1	27
75	Edge physics and H-mode studies in ASDEX Upgrade. Plasma Physics and Controlled Fusion, 1993, 35, B205-B214.	2.1	27
76	Study of gross and net erosion in the ASDEX upgrade divertor. Journal of Nuclear Materials, 1997, 241-243, 684-689.	2.7	27
77	Overview of ASDEX Upgrade results. Nuclear Fusion, 2011, 51, 094012.	3.5	27
78	Synergistic effects by simultaneous bombardment of tungsten with hydrogen and carbon. Journal of Nuclear Materials, 2001, 290-293, 107-111.	2.7	26
79	First studies of ITER-diagnostic mirrors in a tokamak with an all-metal interior: results of the first mirror test in ASDEX Upgrade. Nuclear Fusion, 2013, 53, 073033.	3.5	26
80	The tungsten experiment in ASDEX Upgrade. Journal of Nuclear Materials, 1997, 241-243, 678-683.	2.7	25
81	Depth profile determination with confidence intervals from Rutherford backscattering data. New Journal of Physics, 1999, 1, 11-11.	2.9	25
82	Carbon transport, deposition and fuel accumulation in castellated structures exposed in TEXTOR. Journal of Nuclear Materials, 2007, 367-370, 1481-1486.	2.7	25
83	Controlled tungsten melting and droplet ejection studies in ASDEX Upgrade. Physica Scripta, 2011, T145, 014067.	2.5	25
84	Erosion and migration of tungsten employed at the main chamber first wall of ASDEX Upgrade. Journal of Nuclear Materials, 2003, 313-316, 327-332.	2.7	24
85	Impurity accumulation in plasma regimes with high energy confinement. Journal of Nuclear Materials, 1989, 162-164, 14-23.	2.7	23
86	Relevance of surface roughness to tungsten sputtering and carbon implantation. Journal of Applied Physics, 2006, 100, 113302.	2.5	23
87	Overview of progress in European medium sized tokamaks towards an integrated plasma-edge/wall solution ^a. Nuclear Fusion, 2017, 57, 102014.	3.5	23
88	Determination of impurity transport coefficients by harmonic analysis. Nuclear Fusion, 1990, 30, 2392-2396.	3.5	22
89	Hydrogen retention in ITER relevant mixed material layers. Journal of Nuclear Materials, 2009, 390-391, 659-662.	2.7	21
90	Divertor retention for recycling impurities. Nuclear Fusion, 1992, 32, 1835-1844.	3.5	20

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91	Overview of ASDEX Upgrade results. Nuclear Fusion, 2003, 43, 1570-1582.	3.5	20
92	Overview of the recent DiMES and MiMES experiments in DIII-D. Physica Scripta, 2009, T138, 014007.	2.5	20
93	Overview of material migration and mixing, fuel retention and cleaning of ITER-like castellated structures in TEXTOR. Journal of Nuclear Materials, 2011, 415, S289-S292.	2.7	20
94	Multi machine scaling of fuel retention in 4 carbon dominated tokamaks. Journal of Nuclear Materials, 2011, 415, S735-S739.	2.7	20
95	Transient induced tungsten melting at the Joint European Torus (JET). Physica Scripta, 2017, T170, 014013.	2.5	20
96	Boronization of ASDEX. Journal of Nuclear Materials, 1990, 176-177, 350-356.	2.7	19
97	Investigation of transient melting of tungsten by ELMs in ASDEX Upgrade. Physica Scripta, 2017, T170, 014030.	2.5	19
98	Sustained W-melting experiments on actively cooled ITER-like plasma facing unit in WEST. Physica Scripta, 2021, 96, 124057.	2.5	19
99	Confinement regime transitions in ASDEX. Plasma Physics and Controlled Fusion, 1989, 31, 1629-1648.	2.1	18
100	Studies on 13C deposition in ASDEX Upgrade. Journal of Nuclear Materials, 2005, 337-339, 55-59.	2.7	18
101	Nitrogen retention mechanisms in tokamaks with beryllium and tungsten plasma-facing surfaces. Physica Scripta, 2016, T167, 014077.	2.5	18
102	Surface modification of He pre-exposed tungsten samples by He plasma impact in the divertor manipulator of ASDEX Upgrade. Nuclear Materials and Energy, 2017, 12, 575-581.	1.3	18
103	Simulations of thermionic suppression during tungsten transient melting experiments. Physica Scripta, 2017, T170, 014069.	2.5	18
104	Self-consistent description of the replacement current driving melt layer motion in fusion devices. Nuclear Fusion, 2018, 58, 106003.	3.5	18
105	Long-pulse heating of ASDEX plasmas. Plasma Physics and Controlled Fusion, 1988, 30, 1443-1453.	2.1	17
106	Divertor tokamak operation at high densities on ASDEX Upgrade. Plasma Physics and Controlled Fusion, 1997, 39, B19-B38.	2.1	17
107	A sublimation probe for the injection of high-Z impurities into fusion devices. Review of Scientific Instruments, 1999, 70, 63-67.	1.3	17
108	Carbon chemical erosion in H-mode discharges in ASDEX Upgrade divertor IIb: flux dependence and local redeposition. Journal of Nuclear Materials, 2005, 337-339, 985-989.	2.7	17

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109	DiMES studies of temperature dependence of carbon erosion and re-deposition in the lower divertor of DIII-D under detachment. Physica Scripta, 2007, T128, 29-34.	2.5	17
110	Tungsten sputtering and accumulation of implanted carbon and deuterium by simultaneous bombardment with D and C ions. Journal of Nuclear Materials, 2007, 363-365, 1184-1189.	2.7	17
111	Physics of enhanced confinement with peaked and broad density profiles. Plasma Physics and Controlled Fusion, 1990, 32, 965-981.	2.1	16
112	Experimental investigations of high-Z materials in the ASDEX-Upgrade divertor. Journal of Nuclear Materials, 1995, 220-222, 231-234.	2.7	16
113	Measurements of beryllium sputtering yields at JET. Journal of Nuclear Materials, 2011, 415, S170-S173.	2.7	16
114	Influence of beryllium carbide formation on deuterium retention and release. Journal of Nuclear Materials, 2011, 415, S713-S716.	2.7	16
115	Gross and net erosion of tungsten in the outer strike-point region of ASDEX Upgrade. Physica Scripta, 2016, T167, 014026.	2.5	16
116	Plasma-wall interaction studies in the full-W ASDEX upgrade during helium plasma discharges. Nuclear Fusion, 2017, 57, 066015.	3.5	16
117	Erosion of tungsten coated tiles on the central column of ASDEX Upgrade. Nuclear Fusion, 2000, 40, 1441-1444.	3.5	15
118	Tungsten redistribution patterns in ASDEX Upgrade. Journal of Nuclear Materials, 2005, 337-339, 10-16.	2.7	15
119	Dual beam experiment for simultaneous irradiation of surfaces with ion species of gaseous and solid-state elements. Review of Scientific Instruments, 2006, 77, 043501.	1.3	15
120	Deuterium depth profiling in JT-60U tiles using the D(3He,p)4He resonant nuclear reaction. Journal of Nuclear Materials, 2007, 363-365, 904-909.	2.7	15
121	Investigation of local carbon transport in the ASDEX Upgrade divertor using 13CH4 puffing. Journal of Nuclear Materials, 2009, 390-391, 68-71.	2.7	15
122	Be wall sources and migration in L-mode discharges after Be evaporation in the JET tokamak. Journal of Nuclear Materials, 2009, 390-391, 110-114.	2.7	15
123	3D modeling of the ASDEX Upgrade edge plasma exposed to a localized tungsten source by means of EMC3-Eirene. Journal of Nuclear Materials, 2011, 415, S505-S508.	2.7	15
124	Fuel retention in impurity seeded discharges in JET after Be evaporation. Nuclear Fusion, 2011, 51, 073007.	3.5	15
125	The effect of non-axisymmetric wall geometry on13C transport in ASDEX Upgrade. Nuclear Fusion, 2012, 52, 032001.	3.5	15
126	Global migration of impurities in tokamaks. Plasma Physics and Controlled Fusion, 2013, 55, 124029.	2.1	15

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127	Modelling of impurities in the ASDEX-Upgrade divertor with DIVIMP. Journal of Nuclear Materials, 1995, 220-222, 548-552.	2.7	14
128	Migration of tungsten eroded from divertor tiles in ASDEX Upgrade. Journal of Nuclear Materials, 1997, 241-243, 734-738.	2.7	14
129	DIVIMP simulation of W transport in the SOL of JET H-mode plasmas. Physica Scripta, 2011, T145, 014013.	2.5	14
130	Modeling of tungsten transport in the SOL for sources at the central column of ASDEX Upgrade using DIVIMP. Journal of Nuclear Materials, 2003, 313-316, 1216-1220.	2.7	13
131	Morphology and changes of elemental surface composition of tungsten bombarded with carbon ions. Nuclear Instruments & Methods in Physics Research B, 2008, 266, 1979-1986.	1.4	13
132	Simulations with current constraints of ELM-induced tungsten melt motion in ASDEX Upgrade. Physica Scripta, 2017, T170, 014006.	2.5	13
133	MEMOS 3D modelling of ELM-induced transient melt damage on an inclined tungsten surface in the ASDEX Upgrade outer divertor. Nuclear Materials and Energy, 2018, 17, 194-199.	1.3	13
134	Gross and net erosion balance of plasma-facing materials in full-W tokamaks. Nuclear Fusion, 2021, 61, 116006.	3.5	13
135	Erosion of W and deposition of C due to bombardment with D and CH3. Journal of Nuclear Materials, 1998, 258-263, 912-916.	2.7	12
136	Transition from tungsten erosion to carbon layer deposition with simultaneous bombardment of tungsten by helium and carbon. Journal of Applied Physics, 2007, 101, 104906.	2.5	12
137	Migration and deposition of13C in the full-tungsten ASDEX Upgrade tokamak. Plasma Physics and Controlled Fusion, 2010, 52, 065006.	2.1	12
138	Effect of EÃ—B driven transport on the deposition of carbon in the outer divertor of ASDEX Upgrade. Journal of Nuclear Materials, 2011, 415, S231-S234.	2.7	12
139	Interaction of metal dust adhered on castellated substrates with the ELMy H-mode plasmas of ASDEX-Upgrade. Nuclear Fusion, 2018, 58, 106023.	3.5	12
140	Overview of ASDEX Upgrade results. Nuclear Fusion, 2009, 49, 104009.	3.5	11
141	Deuterium retention in bulk tungsten exposed to the outer divertor plasma of ASDEX Upgrade. Physica Scripta, 2011, T145, 014033.	2.5	11
142	An overview of JET edge modelling activities. Journal of Nuclear Materials, 2003, 313-316, 868-872.	2.7	10
143	Principal processes occurring at simultaneous bombardment of tungsten by carbon and deuterium ions. Journal of Applied Physics, 2007, 102, 074923.	2.5	10
144	Discharge resolved impurity flux measurements in the edge plasma of ASDEX Upgrade by exposure of collector probes. Journal of Nuclear Materials, 2007, 363-365, 242-246.	2.7	10

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145	Divertor and midplane materials evaluation system in DIII-D. Journal of Nuclear Materials, 2007, 363-365, 276-281.	2.7	10
146	Simultaneous irradiation of tungsten with deuterium and carbon at elevated temperatures. Journal of Nuclear Materials, 2009, 390-391, 971-974.	2.7	10
147	Deuterium retention in tungsten used in ASDEX Upgrade: comparison of tokamak and laboratory studies. Physica Scripta, 2014, T159, 014043.	2.5	10
148	Nitrogen retention in ASDEX Upgrade. Journal of Nuclear Materials, 2015, 463, 668-671.	2.7	10
149	Castellated structures for ITER: the influence of the shape of castellation on the impurity deposition and fuel accumulation in gaps. Physica Scripta, 2007, T128, 45-49.	2.5	10
150	Sawtooth-free Ohmic discharges in ASDEX and aspects of neoclassical ion transport. Nuclear Fusion, 1991, 31, 2291-2304.	3.5	9
151	AMS â€“ Sensitive tool used as nuclear safeguard and to diagnose fusion experiments. Nuclear Instruments & Methods in Physics Research B, 2007, 259, 694-701.	1.4	9
152	Ion beam analysis of H and D retention in the near surface layers of JT-60U plasma facing wall tiles. Journal of Nuclear Materials, 2007, 363-365, 949-954.	2.7	9
153	Interpretation of Be migration studies at JET and validation of an integrated numerical model for plasma impurity transport and wall composition dynamics. Journal of Nuclear Materials, 2011, 415, S305-S309.	2.7	9
154	3D trajectories re-construction of droplets ejected in controlled tungsten melting studies in ASDEX Upgrade. Journal of Nuclear Materials, 2013, 438, S846-S851.	2.7	9
155	Nitrogen transport in ASDEX Upgrade: Role of surface roughness and transport to the main wall. Nuclear Materials and Energy, 2017, 12, 51-59.	1.3	9
156	The MEMOS-U macroscopic melt dynamics codeâ€”benchmarking and applications. Physica Scripta, 2021, 96, 124009.	2.5	9
157	Towards an improved understanding of the relationship between plasma edge and materials issues in a next-step fusion device. Journal of Nuclear Materials, 2001, 290-293, 255-259.	2.7	8
158	Erosion and migration of tungsten employed at the central column heat shield of ASDEX Upgrade. Journal of Nuclear Materials, 2002, 307-311, 139-143.	2.7	8
159	Formation of D inventories and structural modifications by deuterium bombardment of tungsten thin films. Journal of Nuclear Materials, 2005, 337-339, 965-969.	2.7	8
160	DIVIMP-B2-EIRENE modelling of 13C migration and deposition in ASDEX Upgrade L-mode plasmas. Journal of Nuclear Materials, 2010, 396, 228-233.	2.7	8
161	Global migration of 13C impurities in high-density L-mode plasmas in ASDEX Upgrade. Journal of Nuclear Materials, 2013, 438, S694-S697.	2.7	8
162	Chemically assisted physical sputtering of Tungsten: Identification via the <mml:math xmlns:mml="http://www.w3.org/1998/Math/MathML" altimg="si1.gif" overflow="scroll"><mml:mrow><mml:msup><mml:mrow /><mml:mn>6</mml:mn></mml:msup><mml:mstyle mathvariant="normal"><mml:mi>Î</mml:mi></mml:mstyle><mml:msup><mml:mo>â†’</mml:mo><mml:mn>6</mml:mn></mml:msup> mathvariant="normal"><mml:mi>Î£</mml:mi></mml:mstyle><mml:mo>+</mml:mo></mml:msup></mml:mrow></mml:math>	1.3	8

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163	Studies of tungsten erosion at the inner and outer main chamber wall of the ASDEX Upgrade tokamak. Journal of Nuclear Materials, 2001, 290-293, 326-330.	2.7	7
164	Chapter 10: Plasma-Wall Interaction and First-Wall Materials in ASDEX Upgrade. Fusion Science and Technology, 2003, 44, 692-707.	1.1	7
165	Castellated structures for ITER: Differences of impurity deposition and fuel accumulation in the toroidal and poloidal gaps. Journal of Nuclear Materials, 2009, 386-388, 809-812.	2.7	7
166	Outer divertor of ASDEX Upgrade in low-density L-mode discharges in forward and reversed magnetic field: II. Analysis of local impurity migration. Nuclear Fusion, 2012, 52, 103007.	3.5	7
167	An overview of sputtering-related processes occurring at mixed surfaces formed by simultaneous C+ and D+ irradiation of W. Journal of Nuclear Materials, 2012, 427, 401-410.	2.7	7
168	Modelling of13CH4injection and local carbon deposition at the outer divertor of ASDEX Upgrade. Physica Scripta, 2009, T138, 014019.	2.5	7
169	Surface morphology influence on deuterium retention in beryllium films prepared by thermionic vacuum arc method. Nuclear Instruments & Methods in Physics Research B, 2009, 267, 426-429.	1.4	6
170	Modelling of Carbon Transport in the Outer Divertor Plasma of ASDEX Upgrade. Contributions To Plasma Physics, 2010, 50, 439-444.	1.1	6
171	DIVIMP simulations of 13C puffing experiments in ASDEX Upgrade L-mode plasma. Journal of Nuclear Materials, 2011, 415, S479-S482.	2.7	6
172	Simulating the nitrogen migration in Be/W tokamaks with WallDYN. Physica Scripta, 2016, T167, 014079.	2.5	6
173	An improved model for the accurate calculation of parallel heat fluxes at the JET bulk tungsten outer divertor. Nuclear Fusion, 2018, 58, 106034.	3.5	6
174	Micro-NRA and micro-3HIXE with 3 He microbeam on samples exposed in ASDEX Upgrade and Pilot-PSI machines. Nuclear Instruments & Methods in Physics Research B, 2017, 404, 179-184.	1.4	5
175	Study of gross and net erosion in the ASDEX Upgrade divertor. Journal of Nuclear Materials, 1997, 241-243, 684-689.	2.7	5
176	Migration of tungsten eroded from divertor tiles in ASDEX Upgrade. Journal of Nuclear Materials, 1997, 241-243, 734-738.	2.7	5
177	Plasmaâ€“wall interaction at the ASDEX Upgrade tungsten heat shield. Fusion Engineering and Design, 2001, 56-57, 189-193.	1.9	4
178	Tritium distribution on plasma-facing tiles from ASDEX Upgrade. Journal of Nuclear Materials, 2005, 337-339, 634-638.	2.7	4
179	Modeling tungsten and carbon sputtering by carbon at elevated temperatures. Physica Scripta, 2009, T138, 014045.	2.5	4
180	Global transport of light elements boron and carbon in the full-W ASDEX Upgrade. Journal of Nuclear Materials, 2011, 415, S227-S230.	2.7	4

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181	First results from the¹⁰Be marker experiment in JET with ITER-like wall. Nuclear Fusion, 2014, 54, 082004.	3.5	4
182	Experimental analysis and WallDYN simulations of the global nitrogen migration in ASDEX Upgrade L-mode discharges. Nuclear Fusion, 2016, 56, 036014.	3.5	4
183	Impact of H-mode plasma operation on pre-damaged tungsten divertor tiles in ASDEX Upgrade. Physica Scripta, 2020, T171, 014037.	2.5	4
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