

Suguru Masuzaki

List of Publications by Year in descending order

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#	ARTICLE	IF	CITATIONS
1	Plasma-surface interaction in the stellarator W7-X: conclusions drawn from operation with graphite plasma-facing components. Nuclear Fusion, 2022, 62, 016006.	3.5	12
2	Recent results from deuterium experiments on the large helical device and their contribution to fusion reactor development. Nuclear Fusion, 2022, 62, 042019.	3.5	25
3	Observation of a reduced-turbulence regime with boron powder injection in a stellarator. Nature Physics, 2022, 18, 350-356.	16.7	19
4	An overview of tritium retention in dust particles from the JET-ILW divertor. Physica Scripta, 2022, 97, 024008.	2.5	3
5	Particle control in long-pulse discharge using divertor pumping in LHD. Physica Scripta, 2022, 97, 035601.	2.5	3
6	Confinement improvement during detached phase with RMP application in deuterium plasmas of LHD. Nuclear Fusion, 2022, 62, 056006.	3.5	3
7	Simulation Analysis of the Carbon Deposition Profile on Directional Material Probes in the Large Helical Device Using the ERO2.0 Code. Plasma and Fusion Research, 2022, 17, 2403010-2403010.	0.7	1
8	Application of High-Frequency Ultrasonic Test to the Non-Destructive Inspection of W-Cu Bonded Interface. Plasma and Fusion Research, 2022, 17, 2405013-2405013.	0.7	2
9	Spatial Profiles of NeVI-NeX Emission in ECR-Heated Discharges of the Large Helical Device with Divertor Detachment Induced by RMP Application and Ne Impurity Seeding. Plasma and Fusion Research, 2022, 17, 2402022-2402022.	0.7	1
10	Real-time wall conditioning and recycling modification utilizing boron and boron nitride powder injections into the Large Helical Device. Nuclear Fusion, 2022, 62, 086021.	3.5	7
11	Experimental observations and modelling of radiation asymmetries during N2 seeding in LHD. Nuclear Materials and Energy, 2021, 26, 100848.	1.3	4
12	EUV and VUV Spectra of NeIII-NeX Line Emissions Observed in the Impurity Gas-Puffing Experiments of the Large Helical Device. Plasma and Fusion Research, 2021, 16, 2402006-2402006.	0.7	3
13	Application of Divertor Pumping to Long-Pulse Discharge for Particle Control in LHD. Plasma and Fusion Research, 2021, 16, 1202014-1202014.	0.7	1
14	Data-Driven Approach on the Mechanism of Radiative Collapse in the Large Helical Device. Plasma and Fusion Research, 2021, 16, 2402010-2402010.	0.7	1
15	Simulation of Impurity Transport and Deposition in the Closed Helical Divertor in the Large Helical Device. Plasma and Fusion Research, 2021, 16, 2403004-2403004.	0.7	2
16	Global distribution of tritium in JET with the ITER-like wall. Nuclear Materials and Energy, 2021, 26, 100930.	1.3	7
17	Investigation of the distribution of remaining tritium in divertor in LHD. Nuclear Materials and Energy, 2021, 26, 100884.	1.3	3
18	Advanced multi-step brazing for fabrication of a divertor heat removal component. Nuclear Fusion, 2021, 61, 046016.	3.5	8

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19	Divertor heat load distribution measurements with infrared thermography in the LHD helical divertor. <i>Fusion Engineering and Design</i> , 2021, 165, 112235.	1.9	2
20	Observation of Arc Trails with Significant Damage due to Glow Discharge Wall Conditioning in the Large Helical Device. <i>Plasma and Fusion Research</i> , 2021, 16, 1202061-1202061.	0.7	0
21	Investigation on tritium retention and surface properties on the first wall in the large helical Device. <i>Nuclear Materials and Energy</i> , 2021, 27, 100906.	1.3	3
22	Isotope effects on transport in LHD. <i>Plasma Physics and Controlled Fusion</i> , 2021, 63, 094001.	2.1	7
23	Demonstration of reduced neoclassical energy transport in Wendelstein 7-X. <i>Nature</i> , 2021, 596, 221-226.	27.8	69
24	Removal of tritium from vacuum vessel by RF heated plasmas in LHD. <i>Physica Scripta</i> , 2021, 96, 124007.	2.5	4
25	Development of the brazing technique of W and JLF-1 by Ni-P filler material. <i>Fusion Engineering and Design</i> , 2021, 170, 112687.	1.9	3
26	Quantitative evaluation of hydrogen retention of solid tin after exposure to hydrogen plasma. <i>Fusion Engineering and Design</i> , 2021, 170, 112532.	1.9	3
27	Steady-state sustainment of divertor detachment with multi-species impurity seeding in LHD. <i>Nuclear Fusion</i> , 2021, 61, 126018.	3.5	3
28	Characteristics of plasma parameters and turbulence in the isotope-mixing and the non-mixing states in hydrogenâ€“deuterium mixture plasmas in the large helical device. <i>Nuclear Fusion</i> , 2021, 61, 016012.	3.5	12
29	Plasma-wall interaction studies in W7-X: main results from the recent divertor operations. <i>Physica Scripta</i> , 2021, 96, 124059.	2.5	10
30	Determination of retained tritium from ILW dust particles in JET. <i>Nuclear Materials and Energy</i> , 2020, 22, 100673.	1.3	7
31	Tritium removal from the LHD first-wall by the hydrogen plasma discharge. <i>Fusion Engineering and Design</i> , 2020, 159, 111879.	1.9	5
32	Fullâ€“torus impurity transport simulation for optimizing plasma discharge operation using a multiâ€“species impurity powder dropper in the large helical device. <i>Contributions To Plasma Physics</i> , 2020, 60, e201900101.	1.1	14
33	First impurity powder injection experiments in LHD. <i>Nuclear Materials and Energy</i> , 2020, 25, 100842.	1.3	17
34	Boron transport simulation using the ERO2.0 code for real-time wall conditioning in the large helical device. <i>Nuclear Materials and Energy</i> , 2020, 25, 100853.	1.3	4
35	Tritium distribution analysis of Be limiter tiles from JET-ITER like wall campaigns using imaging plate technique and β^2 -ray induced X-ray spectrometry. <i>Fusion Engineering and Design</i> , 2020, 160, 111959.	1.9	6
36	Inspection of Arc Trails Formed in Stellarator/Heliotron Devices W7-X and LHD. <i>Plasma and Fusion Research</i> , 2020, 15, 2402012-2402012.	0.7	5

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37	Investigation of remaining tritium in the LHD vacuum vessel after the first deuterium experimental campaign. <i>Physica Scripta</i> , 2020, T171, 014068.	2.5	10
38	Comparison of Hydrogen Isotope Retention in Divertor Tiles of JET with the ITER-Like Wall Following Campaigns in 2011â€“2012 and 2015â€“2016. <i>Fusion Science and Technology</i> , 2020, 76, 439-445.	1.1	3
39	Application of the Advanced Multi-Step Brazing for fabrication of the high heat flux component. <i>Journal of Nuclear Materials</i> , 2020, 538, 152264.	2.7	7
40	Inspection of W 7-X plasma-facing components after the operation phase OP1.2b: observations and first assessments. <i>Physica Scripta</i> , 2020, T171, 014033.	2.5	11
41	In-vessel colorimetry of Wendelstein 7-X first wall components: variation of layer deposition distribution in OP1.2a and OP1.2b. <i>Physica Scripta</i> , 2020, T171, 014054.	2.5	5
42	Hydrogen isotope exchange at the surface of C-W mixed material layer on tungsten by gas exposure. <i>Fusion Engineering and Design</i> , 2020, 157, 111633.	1.9	3
43	Helium and hydrogen interaction in tungsten simultaneously irradiated by He ⁺ -H ₂ ⁺ at high temperature. <i>International Journal of Hydrogen Energy</i> , 2020, 45, 9959-9968.	7.1	7
44	Extended investigations of isotope effects on ECRH plasma in LHD. <i>Plasma Physics and Controlled Fusion</i> , 2020, 62, 024006.	2.1	10
45	Surface morphology of the bulk tungsten divertor tiles from JET ITER-like wall. <i>Physica Scripta</i> , 2020, T171, 014010.	2.5	4
46	Transition between Isotope-Mixing and Nonmixing States in Hydrogen-Deuterium Mixture Plasmas. <i>Physical Review Letters</i> , 2020, 124, 025002.	7.8	18
47	Divertor Detachment with Multi-Species Impurity Seeding in LHD. <i>Plasma and Fusion Research</i> , 2020, 15, 1402051-1402051.	0.7	8
48	Tritium Balance in Large Helical Device during and after the First Deuterium Plasma Experiment Campaign. <i>Plasma and Fusion Research</i> , 2020, 15, 1405062-1405062.	0.7	9
49	Prediction of Radiative Collapse in Large Helical Device Using Feature Extraction by Exhaustive Search. <i>Journal of Fusion Energy</i> , 2020, 39, 500-511.	1.2	3
50	DAMAGING OF PURE TUNGSTEN WITH DIFFERENT MICROSTRUCTURE UNDER SEQUENTIAL QSPA AND LHD PLASMA LOADS. , 2020, , 78-82.		2
51	First divertor physics studies in Wendelstein 7-X. <i>Nuclear Fusion</i> , 2019, 59, 096014.	3.5	34
52	Analysis of mixed-material layers deposited on the toroidal array probes during the FY 2012 LHD plasma campaign. <i>Fusion Engineering and Design</i> , 2019, 147, 111228.	1.9	5
53	Overview of first Wendelstein 7-X high-performance operation. <i>Nuclear Fusion</i> , 2019, 59, 112004.	3.5	165
54	Synergistic effect of nitrogen and hydrogen seeding gases on plasma detachment in the GAMMA 10/PDX tandem mirror. <i>Nuclear Fusion</i> , 2019, 59, 066030.	3.5	18

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55	New approach to the control of particle recycling using divertor pumping in the Large Helical Device. Nuclear Fusion, 2019, 59, 086022.	3.5	8
56	Isotope effects on energy, particle transport and turbulence in electron cyclotron resonant heating plasma of the Large Helical Device. Nuclear Fusion, 2019, 59, 126040.	3.5	16
57	Leak tight joint procedures for ODS-Cu/ODS-Cu by the advanced brazing technique. Fusion Engineering and Design, 2019, 148, 111274.	1.9	8
58	Impact of a resonant magnetic perturbation field on impurity radiation, divertor footprint, and core plasma transport in attached and detached plasmas in the Large Helical Device. Nuclear Fusion, 2019, 59, 096009.	3.5	16
59	Analysis of indefinite divertor footprint with proper orthogonal decomposition in hydrogen/deuterium plasmas in LHD. Nuclear Materials and Energy, 2019, 19, 378-383.	1.3	2
60	Effects of drifts on divertor plasma transport in LHD. Nuclear Materials and Energy, 2019, 18, 281-284.	1.3	10
61	Deformation and fracture behavior of the W/ODS-Cu joint fabricated by the advanced brazing technique. Fusion Engineering and Design, 2019, 146, 1733-1736.	1.9	9
62	Demonstration of suppression of dust generation and partial reduction of the hydrogen retention by tungsten coated graphite divertor tiles in LHD. Nuclear Materials and Energy, 2019, 18, 23-28.	1.3	7
63	The isotope effect on impurities and bulk ion particle transport in the Large Helical Device. Nuclear Fusion, 2019, 59, 056029.	3.5	13
64	Influence of thermal shocks on the He induced surface morphology on tungsten. Nuclear Materials and Energy, 2019, 18, 321-325.	1.3	3
65	New installation of in-vessel Non Evaporable Getter (NEG) pumps for the divertor pump in the LHD. Fusion Engineering and Design, 2019, 143, 226-232.	1.9	6
66	Spectroscopic studies on the enhanced radiation with high Z rare gas seeding for mitigating divertor heat loads in LHD plasmas. Nuclear Materials and Energy, 2019, 19, 195-199.	1.3	2
67	Tritium distributions on W-coated divertor tiles used in the third JET ITER-like wall campaign. Nuclear Materials and Energy, 2019, 18, 258-261.	1.3	10
68	Impact of additional plasma heating on detached plasma formation in divertor simulation experiments using the GAMMA 10/PDX tandem mirror. Nuclear Materials and Energy, 2019, 18, 216-221.	1.3	8
69	Erosion and deposition investigations on Wendelstein 7-X first wall components for the first operation phase in divertor configuration. Fusion Engineering and Design, 2019, 146, 242-245.	1.9	17
70	First Application of 3D Peripheral Plasma Transport Code EMC3-EIRENE to Heliotron J. Plasma and Fusion Research, 2019, 14, 3403127-3403127.	0.7	6
71	Plasma-wall interaction on the divertor tiles of JET ITER-like wall from the viewpoint of micro/nanoscale observations. Fusion Engineering and Design, 2018, 136, 199-204.	1.9	5
72	Establishment of a low recycling state with full density control by active pumping of the closed helical divertor at LHD. Nuclear Fusion, 2018, 58, 014005.	3.5	7

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73	Helium retention behavior in simultaneously He+H2+ irradiated tungsten. Journal of Nuclear Materials, 2018, 502, 289-294.	2.7	15
74	Dust generation in tokamaks: Overview of beryllium and tungsten dust characterisation in JET with the ITER-like wall. Fusion Engineering and Design, 2018, 136, 579-586.	1.9	52
75	Tritium retention characteristics in dust particles in JET with ITER-like wall. Nuclear Materials and Energy, 2018, 17, 279-283.	1.3	20
76	Core plasma confinement during detachment transition with RMP application in LHD. Nuclear Materials and Energy, 2018, 17, 137-141.	1.3	7
77	Impurity transport simulation in the peripheral plasma in the large helical device with tungsten closed helical divertor. Nuclear Materials and Energy, 2018, 17, 188-193.	1.3	3
78	Characterized divertor footprint profile modification with the edge pressure gradient in the Large Helical Device. Plasma Physics and Controlled Fusion, 2018, 60, 125001.	2.1	4
79	Three-dimensional impurity transport modeling of neon-seeded and nitrogen-seeded LHD plasmas. Plasma Physics and Controlled Fusion, 2018, 60, 084005.	2.1	39
80	Heat loading behavior and thermomechanical analyses on plasma spray tungsten coated reduced-activation ferritic/martensitic steel. Fusion Engineering and Design, 2018, 136, 1624-1628.	1.9	4
81	Deuterium retention behavior in simultaneously He+D2+ implanted tungsten. Nuclear Materials and Energy, 2018, 16, 76-81.	1.3	7
82	Correlation of surface chemical states with hydrogen isotope retention in divertor tiles of JET with ITER-Like Wall. Fusion Engineering and Design, 2018, 132, 24-28.	1.9	15
83	Investigation of dust shielding effect of intrinsic ergodic magnetic field line structures in the peripheral plasma in the large helical device. Contributions To Plasma Physics, 2018, 58, 616-621.	1.1	1
84	Maintainability of the helical reactor FFHR-c1 equipped with the liquid metal divertor and cartridge-type blankets. Fusion Engineering and Design, 2018, 136, 1278-1285.	1.9	10
85	Application of EMC3-EIRENE to Estimation of Influence of a Liquid Metal Limiter on an LHD-Type Fusion Plasma. Plasma and Fusion Research, 2018, 13, 3403034-3403034.	0.7	4
86	Temperature impact on the micro structure of tungsten exposed to He irradiation in LHD. Journal of Nuclear Materials, 2017, 484, 24-29.	2.7	14
87	Damage and deuterium retention of re-solidified tungsten following vertical displacement event-like heat load. Nuclear Materials and Energy, 2017, 12, 1303-1307.	1.3	7
88	Toroidally symmetric/asymmetric effect on the divertor flux due to neon/nitrogen seeding in LHD. Nuclear Materials and Energy, 2017, 12, 241-246.	1.3	26
89	Initial growth phase of W-fuzz formation in ultra-long pulse helium discharge in LHD. Nuclear Materials and Energy, 2017, 12, 1358-1362.	1.3	10
90	Effects of modified surfaces produced at plasma-facing surface on hydrogen release behavior in the LHD. Nuclear Materials and Energy, 2017, 12, 483-487.	1.3	5

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91	Fabrication of divertor mock-up with ODS-Cu and W by the improved brazing technique. Nuclear Fusion, 2017, 57, 076009.	3.5	17
92	Micro-/nano-characterization of the surface structures on the divertor tiles from JET ITER-like wall. Fusion Engineering and Design, 2017, 116, 1-4.	1.9	14
93	Two conceptual designs of helical fusion reactor FFHR-d1A based on ITER technologies and challenging ideas. Nuclear Fusion, 2017, 57, 086046.	3.5	34
94	Extension of the operational regime of the LHD towards a deuterium experiment. Nuclear Fusion, 2017, 57, 102023.	3.5	116
95	Conceptual design of a liquid metal limiter/divertor system for the FFHR-d1. Fusion Engineering and Design, 2017, 125, 227-238.	1.9	33
96	Overview of spherical tokamak research in Japan. Nuclear Fusion, 2017, 57, 102005.	3.5	6
97	Simulation of impurity transport in the peripheral plasma due to the emission of dust in long pulse discharges on the Large Helical Device. Nuclear Materials and Energy, 2017, 12, 779-785.	1.3	7
98	Investigation of heat flux deposition on divertor target on the Large Helical Device with EMC3-EIRENE modelling. Plasma Physics and Controlled Fusion, 2017, 59, 085013.	2.1	7
99	Molecular activated recombination in divertor simulation plasma on GAMMA 10/PDX. Nuclear Materials and Energy, 2017, 12, 1004-1009.	1.3	32
100	Influence of mixed material layer formation on hydrogen isotope and He retentions in W exposed to 2014 LHD experiment campaign. Fusion Engineering and Design, 2017, 125, 458-462.	1.9	4
101	Wide-range evaluation of the deposition layer thickness distribution on the first wall by reflection coefficient measurements. Nuclear Materials and Energy, 2017, 12, 1219-1223.	1.3	11
102	The role of the graphite divertor tiles in helium retention on the LHD wall. Nuclear Materials and Energy, 2017, 13, 58-62.	1.3	5
103	Recent progress of divertor simulation research using the GAMMA 10/PDX tandem mirror. Nuclear Fusion, 2017, 57, 116033.	3.5	56
104	Examinations for leak tightness of actively cooled components in ITER and fusion devices. Physica Scripta, 2017, T170, 014045.	2.5	6
105	Development of H, D, T Simultaneous TDS Measurement System and H, D, T Retention Behavior for DT Gas Exposed Tungsten Installed in LHD Plasma Campaign. Fusion Science and Technology, 2017, 71, 351-356.	1.1	1
106	Effects of Mild Baking on Hydrogen Removal from the Modified Surface of the First Wall in the LHD. Plasma and Fusion Research, 2017, 12, 1302048-1302048.	0.7	0
107	Analyses of microstructure, composition and retention of hydrogen isotopes in divertor tiles of JET with the ITER-like wall. Physica Scripta, 2017, T170, 014031.	2.5	13
108	Preparation of erosion and deposition investigations on plasma facing components in Wendelstein 7-X. Physica Scripta, 2017, T170, 014010.	2.5	10

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109	Tritium analysis of divertor tiles used in JET ITER-like wall campaigns by means of I^2 -ray induced x-ray spectrometry. Physica Scripta, 2017, T170, 014014.	2.5	6
110	ACT2: a High Heat Flux Test Facility Using Electron Beam for Fusion Application. Plasma and Fusion Research, 2016, 11, 2405089-2405089.	0.7	14
111	Observation of the inward propagation of spontaneous toroidal flow from the plasma boundary in LHD. Physics of Plasmas, 2016, 23, .	1.9	5
112	Dynamics of three-dimensional radiative structures during RMP assisted detached plasmas on the large helical device and its comparison with EMC3-EIRENE modeling. Nuclear Fusion, 2016, 56, 046002.	3.5	13
113	Progress of long pulse discharges by ECH in LHD. Nuclear Fusion, 2016, 56, 046005.	3.5	7
114	Plasma detachment study of high density helium plasmas in the Pilot-PSI device. Nuclear Fusion, 2016, 56, 126006.	3.5	23
115	Effect of impurity deposition layer formation on D retention in LHD plasma exposed W. Nuclear Materials and Energy, 2016, 9, 84-88.	1.3	9
116	Influence of carbon-dominated deposition layer on He retention and desorption in tungsten. Fusion Engineering and Design, 2016, 112, 117-122.	1.9	8
117	Investigation of arcing on fiber-formed nanostructured tungsten by pulsed plasma during steady state plasma irradiation. Fusion Engineering and Design, 2016, 112, 156-161.	1.9	21
118	Design Status of the Structural Components of the Helical Fusion Reactor FFHR-d1. Plasma and Fusion Research, 2016, 11, 2405061-2405061.	0.7	2
119	Simulation Analysis of Carbon Deposition Profile in the Closed Helical Divertor Configuration in the Large Helical Device. Contributions To Plasma Physics, 2016, 56, 651-656.	1.1	4
120	Mie-Scattering Ellipsometry System for Analysis of Dust Formation Process in Large Plasma Device. IEEE Transactions on Plasma Science, 2016, 44, 1032-1035.	1.3	8
121	Effect of neutral hydrogen on edge impurity behavior in stochastic magnetic field layer of Large Helical Device. Journal of Nuclear Materials, 2015, 463, 644-648.	2.7	4
122	Vacuum ultraviolet spectroscopy in detached plasmas with impurity gas seeding in LHD. Journal of Nuclear Materials, 2015, 463, 561-564.	2.7	3
123	Progress of divertor simulation research toward the realization of detached plasma using a large tandem mirror device. Journal of Nuclear Materials, 2015, 463, 537-540.	2.7	18
124	Potential of Copper Alloys using a Divertor Heat Sink in the Helical Reactor FFHR-d1 and their Brazing Properties with Tungsten Armor by using the Typical Candidate Filler Materials. Plasma and Fusion Research, 2015, 10, 3405035-3405035.	0.7	12
125	Preliminary Examination of Reflection Coefficient Measurement of RGB Lights on the First Wall in LHD. Plasma and Fusion Research, 2015, 10, 1202074-1202074.	0.7	5
126	3D effects of edge magnetic field configuration on divertor/scrape-off layer transport and optimization possibilities for a future reactor. Nuclear Fusion, 2015, 55, 104021.	3.5	23

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127	Real-time mass measurement of dust particles deposited on vessel wall in a divertor simulator using quartz crystal microbalances. Journal of Nuclear Materials, 2015, 463, 865-868.	2.7	2
128	Analysis of the three-dimensional trajectories of dusts observed with a stereoscopic fast framing camera in the Large Helical Device. Journal of Nuclear Materials, 2015, 463, 861-864.	2.7	6
129	Effect of the RF wall conditioning on the high performance plasmas in the Large Helical Device. Journal of Nuclear Materials, 2015, 463, 1100-1103.	2.7	10
130	Impact of 3D magnetic field structure on boundary and divertor plasmas in stellarator/heliotron devices. Journal of Nuclear Materials, 2015, 463, 2-10.	2.7	6
131	Plasma wall interaction in long-pulse helium discharge in LHD – Microscopic modification of the wall surface and its impact on particle balance and impurity generation. Journal of Nuclear Materials, 2015, 463, 91-98.	2.7	35
132	Evolution of radiation profiles during detached plasmas and radiative collapse in LHD. Journal of Nuclear Materials, 2015, 463, 551-554.	2.7	1
133	Novel divertor design to mitigate neutron irradiation in the helical reactor FFHR-d1. Fusion Engineering and Design, 2015, 98-99, 1629-1633.	1.9	10
134	Studies of dust transport in long pulse plasma discharges in the large helical device. Nuclear Fusion, 2015, 55, 053014.	3.5	22
135	Development of impurity seeding and radiation enhancement in the helical divertor of LHD. Nuclear Fusion, 2015, 55, 083016.	3.5	21
136	Overview of transport and MHD stability study: focusing on the impact of magnetic field topology in the Large Helical Device. Nuclear Fusion, 2015, 55, 104018.	3.5	10
137	Radiated power distributions in impurity-seeded plasmas in LHD. Journal of Nuclear Materials, 2015, 463, 640-643.	2.7	8
138	Global helium particle balance in LHD. Journal of Nuclear Materials, 2015, 463, 1080-1083.	2.7	22
139	Multi-pin Langmuir probe measurement for identification of blob propagation characteristics in the Large Helical Device. Journal of Nuclear Materials, 2015, 463, 761-764.	2.7	7
140	Two-dimensional study of edge impurity transport in the Large Helical Device. Plasma Physics and Controlled Fusion, 2014, 56, 094007.	2.1	14
141	First EMC3-EIRENE Simulations with Divertor Legs of LHD in Realistic Device Geometry. Contributions To Plasma Physics, 2014, 54, 437-441.	1.1	45
142	A New Deduction Method of Heat Flux Evolution From Thermal Probe Data. Contributions To Plasma Physics, 2014, 54, 285-290.	1.1	3
143	Mitigation of large amplitude edge-localized modes by resonant magnetic perturbations on LHD. Nuclear Fusion, 2014, 54, 033001.	3.5	9
144	Design of structural components for the helical reactor FFHR-d1A. Fusion Engineering and Design, 2014, 89, 2336-2340.	1.9	12

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145	Analysis of radiation environment at divertor in helical reactor FFHR-d1. Fusion Engineering and Design, 2014, 89, 1939-1943.	1.9	17
146	Comparison of hydrogen isotope retention for tungsten probes in LHD vacuum vessel during the experimental campaigns in 2011 and 2012. Fusion Engineering and Design, 2014, 89, 1091-1095.	1.9	8
147	Measured and Modeled Radiation Profiles for LHD Plasma -ÂA Comparison during Plasma Detachment. Plasma and Fusion Research, 2014, 9, 3402064-3402064.	0.7	3
148	Simulation Analysis of Dust-Particle Transport in the Peripheral Plasma in the Large Helical Device. Plasma and Fusion Research, 2014, 9, 3403132-3403132.	0.7	2
149	Contribution of H2plasma etching to radial profile of amount of dust particles in a divertor simulator. Journal of Physics: Conference Series, 2014, 518, 012009.	0.4	0
150	Kinetic effect of high energy ions on the temperature profile in the boundary plasma region. Journal of Nuclear Materials, 2013, 438, S472-S474.	2.7	2
151	Evaluation of the surface morphologies and erosion/deposition profiles on the LHD first-wall by using toroidal array probes. Journal of Nuclear Materials, 2013, 442, S873-S879.	2.7	7
152	Recent results of divertor simulation research using an end-cell of a large tandem mirror device. Journal of Nuclear Materials, 2013, 438, S738-S741.	2.7	25
153	Study of hydrogen isotopes behavior in carbon based materials with in situ ion beam analysis under plasma exposure. Journal of Nuclear Materials, 2013, 438, S1036-S1039.	2.7	4
154	Control of 3D edge radiation structure with resonant magnetic perturbation fields applied to the stochastic layer and stabilization of radiative divertor plasma in LHD. Nuclear Fusion, 2013, 53, 093032.	3.5	48
155	Influence of the resonant magnetic perturbations on transport in the Large Helical Device. Nuclear Fusion, 2013, 53, 113012.	3.5	10
156	Radial-build design and support system for the helical DEMO reactor FFHR-d1. Fusion Engineering and Design, 2013, 88, 2033-2037.	1.9	8
157	Intermittent transport in edge plasma with a 3-D magnetic geometry in the Large Helical Device. Journal of Nuclear Materials, 2013, 438, S563-S566.	2.7	3
158	Edge and divertor plasma measurements with ion sensitive and Mach probes in LHD. Journal of Nuclear Materials, 2013, 438, S1228-S1231.	2.7	4
159	Enhancement of hydrogen isotope retention in tungsten exposed to LHD plasmas. Journal of Nuclear Materials, 2013, 438, S1055-S1058.	2.7	9
160	Removal of carbon deposited film and hydrogen retention control by low temperature Hâ€“Câ€“N reactive plasmas. Journal of Nuclear Materials, 2013, 438, S1092-S1095.	2.7	3
161	Discharge power dependence of carbon dust flux in a divertor simulator. Journal of Nuclear Materials, 2013, 438, S788-S791.	2.7	5
162	Development of high-grade VPS-tungsten coatings on F82H reduced activation steel. Journal of Nuclear Materials, 2013, 442, S287-S291.	2.7	11

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163	Microstructural characterization of mixed-material deposition layer on the LHD divertor tiles by using nano-geological diagnosis. Journal of Nuclear Materials, 2013, 438, S818-S821.	2.7	16
164	Divertor heat and particle control experiments on the large helical device. Journal of Nuclear Materials, 2013, 438, S133-S138.	2.7	13
165	Effect of a baffle divertor structure on neutral hydrogen and helium transport in the Large Helical Device. Journal of Nuclear Materials, 2013, 438, S559-S562.	2.7	3
166	Enhancement of hydrogen isotope retention capacity for the impurity deposited tungsten by long-term plasma exposure in LHD. Fusion Engineering and Design, 2013, 88, 1699-1703.	1.9	13
167	Effects of DC substrate bias voltage on dust flux in the Large Helical Device. Journal of Nuclear Materials, 2013, 438, S727-S730.	2.7	5
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169	Flux Control of Carbon Nanoparticles Generated due to Interactions between Hydrogen Plasmas and Graphite Using DC-Biased Substrates. Japanese Journal of Applied Physics, 2013, 52, 11NA08.	1.5	2
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