

# Per Petersson

## List of Publications by Year in descending order

Source: <https://exaly.com/author-pdf/7091340/publications.pdf>

Version: 2024-02-01

48  
papers

1,181  
citations

361413

20  
h-index

395702

33  
g-index

49  
all docs

49  
docs citations

49  
times ranked

906  
citing authors

#	ARTICLE	IF	CITATIONS
1	Application of Ion Beam Analysis in Studies of First Wall Materials in Controlled Fusion Devices. <i>Physics</i> , 2022, 4, 37-50.	1.4	0
2	Isotope removal experiment in JET-ILW in view of T-removal after the 2nd DT campaign at JET. <i>Physica Scripta</i> , 2022, 97, 044001.	2.5	7
3	3D deposition patterns of deuterium retention and impurities in the COMPASS divertor: a data-driven root cause analysis and prediction approach. <i>Fusion Engineering and Design</i> , 2022, 179, 113118.	1.9	1
4	The upgraded TOMAS device: A toroidal plasma facility for wall conditioning, plasma production, and plasma-surface interaction studies. <i>Review of Scientific Instruments</i> , 2021, 92, 023506.	1.3	13
5	First mirror erosion-deposition studies in JET using an ITER-like mirror test assembly. <i>Nuclear Fusion</i> , 2021, 61, 046022.	3.5	13
6	Characterization of neutral particle fluxes from ICWC and ECWC plasmas in the TOMAS facility. <i>Physica Scripta</i> , 2021, 96, 124025.	2.5	7
7	Fuel retention and erosion-deposition on inner wall cladding tiles in JET-ILW. <i>Physica Scripta</i> , 2021, 96, 124071.	2.5	7
8	Evaluation of tritium retention in plasma facing components during JET tritium operations. <i>Physica Scripta</i> , 2021, 96, 124075.	2.5	14
9	Ion beam analysis of fusion plasma-facing materials and components: facilities and research challenges. <i>Nuclear Fusion</i> , 2020, 60, 025001.	3.5	54
10	Comparison of erosion and deposition in JET divertor during the first three ITER-like wall campaigns. <i>Physica Scripta</i> , 2020, T171, 014059.	2.5	19
11	Fuel inventory and impurity deposition in castellated tungsten tiles in KSTAR: experiment and modelling. <i>Physica Scripta</i> , 2020, T171, 014049.	2.5	2
12	Deuterium as a cleaning gas for ITER first mirrors: experimental study on beryllium deposits from laboratory and JET-ILW. <i>Nuclear Fusion</i> , 2019, 59, 096027.	3.5	15
13	Beryllium melting and erosion on the upper dump plates in JET during three ITER-like wall campaigns. <i>Nuclear Fusion</i> , 2019, 59, 086009.	3.5	45
14	First mirror test in JET for ITER: Complete overview after three ILW campaigns. <i>Nuclear Materials and Energy</i> , 2019, 19, 59-66.	1.3	24
15	Analysis of deposited layers with deuterium and impurity elements on samples from the divertor of JET with ITER-like wall. <i>Journal of Nuclear Materials</i> , 2019, 516, 202-213.	2.7	18
16	Ion Microbeam Analyses of Dust Particles and Codeposits from JET with the ITER-Like Wall. <i>Analytical Chemistry</i> , 2018, 90, 5744-5752.	6.5	12
17	Dust generation in tokamaks: Overview of beryllium and tungsten dust characterisation in JET with the ITER-like wall. <i>Fusion Engineering and Design</i> , 2018, 136, 579-586.	1.9	52
18	Review on global migration, fuel retention and modelling after TEXTOR decommission. <i>Nuclear Materials and Energy</i> , 2018, 17, 83-112.	1.3	9

#	ARTICLE	IF	CITATIONS
19	Plasma impact on diagnostic mirrors in JET. Nuclear Materials and Energy, 2017, 12, 506-512.	1.3	25
20	Design of an ICRF system for plasma-wall interactions and RF plasma production studies on TOMAS. Fusion Engineering and Design, 2017, 123, 317-320.	1.9	4
21	Overview of the JET ITER-like wall divertor. Nuclear Materials and Energy, 2017, 12, 499-505.	1.3	46
22	Microanalysis of deposited layers in the inner divertor of JET with ITER-like wall. Nuclear Materials and Energy, 2017, 12, 412-417.	1.3	5
23	Fuel inventory and deposition in castellated structures in JET-ILW. Nuclear Fusion, 2017, 57, 066027.	3.5	25
24	Investigation of probe surfaces after ion cyclotron wall conditioning in ASDEX upgrade. Nuclear Materials and Energy, 2017, 12, 733-735.	1.3	3
25	Overview of wall probes for erosion and deposition studies in the TEXTOR tokamak. Matter and Radiation at Extremes, 2017, 2, 87-104.	3.9	23
26	Overview of fuel inventory in JET with the ITER-like wall. Nuclear Fusion, 2017, 57, 086045.	3.5	47
27	Impurity re-distribution in the corner regions of the JET divertor. Physica Scripta, 2017, T170, 014060.	2.5	6
28	Fine metal dust particles on the wall probes from JET-ILW. Physica Scripta, 2017, T170, 014038.	2.5	22
29	Interaction of candidate plasma facing materials with tokamak plasma in COMPASS. Journal of Nuclear Materials, 2017, 493, 102-119.	2.7	5
30	Metallic mirrors for plasma diagnosis in current and future reactors: tests for ITER and DEMO. Physica Scripta, 2017, T170, 014061.	2.5	12
31	A combined segmented anode gas ionization chamber and time-of-flight detector for heavy ion elastic recoil detection analysis. Review of Scientific Instruments, 2016, 87, 103303.	1.3	98
32	Long-term fuel retention in JET ITER-like wall. Physica Scripta, 2016, T167, 014075.	2.5	52
33	Plasma cleaning of beryllium coated mirrors. Physica Scripta, 2016, T167, 014069.	2.5	24
34	Beryllium migration in JET ITER-like wall plasmas. Nuclear Fusion, 2015, 55, 063021.	3.5	83
35	Co-deposited layers in the divertor region of JET-ILW. Journal of Nuclear Materials, 2015, 463, 814-817.	2.7	32
36	Combined ion micro probe and SEM analysis of strongly non uniform deposits in fusion devices. Nuclear Instruments & Methods in Physics Research B, 2015, 342, 19-28.	1.4	13

#	ARTICLE	IF	CITATIONS
37	An overview of the comprehensive First Mirror Test in JET with ITER-like wall. Physica Scripta, 2014, T159, 014011.	2.5	59
38	Overview of nitrogen-15 application as a tracer gas for material migration and retention studies in tokamaks. Physica Scripta, 2014, T159, 014042.	2.5	12
39	Impact of ion cyclotron wall conditioning on fuel removal from plasma-facing components at TEXTOR. Physica Scripta, 2014, T159, 014017.	2.5	9
40	Laser-assisted cleaning of beryllium-containing mirror samples from JET and PISCES-B. Fusion Engineering and Design, 2014, 89, 122-130.	1.9	23
41	Self-consistent application of ion cyclotron wall conditioning for co-deposited layer removal and recovery of tokamak operation on TEXTOR. Nuclear Fusion, 2013, 53, 123001.	3.5	15
42	Injection of nitrogen-15 tracer into ASDEX-Upgrade: New technique in material migration studies. Journal of Nuclear Materials, 2013, 438, S616-S619.	2.7	19
43	Deuterium inventory in Tore Supra: Coupled carbon-deuterium balance. Journal of Nuclear Materials, 2013, 438, S120-S125.	2.7	38
44	Development of laser-based techniques for <i>in situ</i> characterization of the first wall in ITER and future fusion devices. Nuclear Fusion, 2013, 53, 093002.	3.5	99
45	Nuclear reaction and heavy ion ERD analysis of wall materials from controlled fusion devices: Deuterium and nitrogen-15 studies. Nuclear Instruments & Methods in Physics Research B, 2012, 273, 113-117.	1.4	20
46	Nitrogen and neon retention in plasma-facing materials. Journal of Nuclear Materials, 2011, 415, S223-S226.	2.7	34
47	An overview of nuclear micro-beam analysis of surface and bulk fuel retention in carbon-fibre composites from Tore Supra. Journal of Nuclear Materials, 2011, 415, S761-S764.	2.7	8
48	Dust generation and accumulation in JET-ILW: morphology and stability of co-deposits on main plasma-facing components and wall probes. Physica Scripta, 0, , .	2.5	8