Michael Rieth

List of Publications by Year
in descending order

180	7,130	53794	66911
papers	citations	45 h-index	78 g-index

186	186	186	3221
all docs	docs citations	times ranked	citing authors

#	Article	IF	Citations
1	Materials for in-vessel components. Fusion Engineering and Design, 2022, 174, 112994.	1.9	23
2	Charpy impact tests of tungsten fiberâ€“reinforced composite from â^'150 Â°C to 1000 Â°C. Materials Letters, 2022, 311, 131526.	2.6	3
3	On the mechanical alloying of novel austenitic dual-precipitation strengthened steels. Materials and Design, 2022, 213, 110316.	7.0	1
4	Recent progress in the assessment of irradiation effects for in-vessel fusion materials: tungsten and copper alloys. Nuclear Fusion, 2022, 62, 026045.	3.5	15
5	Effect of neutron irradiation on ductility of tungsten foils developed for tungsten-copper laminates. Nuclear Materials and Energy, 2022, 30, 101133.	1.3	3
6	Nanoscale insights into the corrosion of EUROFER by lithium ceramics. Corrosion Science, 2022, 199, 110190.	6.6	3
7	S/TEM examination and nanomechanical response of W-Eurofer joints brazed with Cu interlayers. Nuclear Materials and Energy, 2022, 31, 101155.	1.3	1
8	Innovative 1000K sodium loop for qualification of new materials for applications in CSP field. AIP Conference Proceedings, 2022, , .	0.4	1
9	Tungstenâ€“tantalum alloys for fusion reactor applications. Journal of Nuclear Materials, 2022, 566, 153740.	2.7	15
10	Plastic deformation in advanced tungsten-based alloys for fusion applications studied by mechanical testing and TEM. International Journal of Refractory Metals and Hard Materials, 2021, 95, 105409.	3.8	13
11	Mechanical properties of tungsten: Recent research on modified tungsten materials in Japan. Journal of Nuclear Materials, 2021, 543, 152506.	2.7	55
12	Comparison of K-doped and pure cold-rolled tungsten sheets: Tensile properties and brittle-to-ductile transition temperatures. Journal of Nuclear Materials, 2021, 544, 152664.	2.7	19
13	New insights into microstructure of neutron-irradiated tungsten. Scientific Reports, 2021, 11, 7572.	3.3	36
14	Additive manufacturing technologies for EUROFER97 components. Journal of Nuclear Materials, 2021, 548, 152859.	2.7	7
15	Impact of materials technology on the breeding blanket design â€“ Recent progress and case studies in materials technology. Fusion Engineering and Design, 2021, 166, 112275.	1.9	6
16	Advances in Additive Manufacturing of fusion materials. Fusion Engineering and Design, 2021, 167, 112309.	1.9	9
17	Technology readiness assessment of materials for DEMO in-vessel applications. Journal of Nuclear Materials, 2021, 550, 152906.	2.7	12
18	Technological aspects in blanket design: Effects of micro-alloying and thermo-mechanical treatments of EUROFER97 type steels after neutron irradiation. Fusion Engineering and Design, 2021, 168, 112645.	1.9	10

#	Article	IF	Citations
19	Microstructure and hardening induced by neutron irradiation in single crystal, ITER specification and cold rolled tungsten. International Journal of Refractory Metals and Hard Materials, 2021, 98, 105522.	3.8	27
20	Neutron irradiation tolerance of potassium-doped and rhenium-alloyed tungsten. Journal of Nuclear Materials, 2021, 553, 153009.	2.7	21
21	DEMO structural materials qualification and development. Fusion Engineering and Design, 2021, 170, 112513.	1.9	8
22	Fabrication routes for advanced first wall designÂalternatives. Nuclear Fusion, 2021, 61, 116067.	3.5	7
23	Microstructure and precipitation behavior of advanced RAFM steels for high-temperature applications on fusion reactors. Materials Characterization, 2021, 180, 111443.	4.4	7
24	Evolution of microstructure in neutron irradiated cold rolled tungsten and its correlation with hardness. Fusion Engineering and Design, 2021, 172, 112784.	1.9	12
25	Microstructural features in additively manufactured EUROFER97 components. Fusion Engineering and Design, 2021, 173, 112813.	1.9	1
26	Post-irradiation microstructural examination of EUROFER-ODS steel irradiated at 300Â°C and 400Â°C. Journal of Nuclear Materials, 2021, 557, 153259.	2.7	5
27	Experimental Investigation of EU-DEMO Breeding Blanket First Wall Mock-Ups in Support of the Manufacturing and Material Development Programmes. Energies, 2021, 14, 7580.	3.1	2
28	Irradiation hardening and ductility loss of Eurofer97 steel variants after neutron irradiation to ITER-TBM relevant conditions. Fusion Engineering and Design, 2021, 173, 112935.	1.9	5
29	Technological Processes for Steel Applications in Nuclear Fusion. Applied Sciences (Switzerland), 2021, 11, 11653.	2.5	9
30	Tungsten modified by potassium doping and rhenium addition for fusion reactor applications. Fusion Engineering and Design, 2020, 152, 111445.	1.9	33
31	Cavity formation and hardness change in He implanted EUROFER97 and EU-ODS EUROFER. Nuclear Materials and Energy, 2020, 22, 100717.	1.3	1
32	The brittle-to-ductile transition in cold-rolled tungsten sheets: On the loss of room-temperature ductility after annealing and the phenomenon of 45Â° embrittlement. International Journal of Refractory Metals and Hard Materials, 2020, 93, 105347.	3.8	11
33	The brittle-to-ductile transition in cold-rolled tungsten sheets: Contributions of grain and subgrain boundaries to the enhanced ductility after pre-deformation. Nuclear Materials and Energy, 2020, 25, 100769.	1.3	4
34	Fabrication of HCPB breeding blanket components using the additive manufacturing processes of selective laser melting and cold spray. Fusion Engineering and Design, 2020, 160, 112026.	1.9	4
35	Correlation of microstructural and mechanical properties of neutron irradiated EUROFER97 steel. Journal of Nuclear Materials, 2020, 538, 152231.	2.7	28
36	The brittle-to-ductile transition in cold-rolled tungsten sheets: the rate-limiting mechanism of plasticity controlling the BDT in ultrafine-grained tungsten. Journal of Materials Science, 2020, 55, 12314-12337.	3.7	18

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37	Elucidating the microstructure of tungsten composite materials produced by powder injection molding. Nuclear Materials and Energy, 2020, 24, 100766.	1.3	3
38	Fracture behavior of tungsten-based composites exposed to steady-state/transient hydrogen plasma. Nuclear Fusion, 2020, 60, 046029.	3.5	13
39	Deformation behaviour and microstructural evolution of EUROFER97-2 under low cycle fatigue conditions. Materials Characterization, 2019, 158, 109943.	4.4	8
40	Effects of neutron irradiation on the brittle to ductile transition in single crystal tungsten. Journal of Nuclear Materials, 2019, 527, 151799.	2.7	34
41	Tensile and impact properties of tungsten-rhenium alloy for plasma-facing components in fusion reactor. Fusion Engineering and Design, 2019, 148, 111323.	1.9	27
42	Microstructural investigation of an extruded austenitic oxide dispersion strengthened steel containing a carbon-containing process control agent. Journal of Nuclear Materials, 2019, 516, 335-346.	2.7	13
43	Improvement of impact properties of tungsten by potassium doping. Fusion Engineering and Design, 2019, 140, 48-61.	1.9	24
44	Manufacturing, high heat flux testing and post mortem analyses of a W-PIM mock-up. Nuclear Materials and Energy, 2019, 20, 100688.	1.3	3
45	Long-term stability of the microstructure of austenitic ODS steel rods produced with a carbon-containing process control agent. Journal of Nuclear Materials, 2019, 523, 111-120.	2.7	6
46	High pulse number thermal shock testing of tungsten alloys produced by powder injection molding. Nuclear Materials and Energy, 2019, 20, 100680.	1.3	10
47	Mechanical properties and microstructure characterization of Eurofer97 steel variants in EUROfusion program. Fusion Engineering and Design, 2019, 146, 2227-2232.	1.9	20
48	Behavior of tungsten under irradiation and plasma interaction. Journal of Nuclear Materials, 2019, 519, 334-368.	2.7	129
49	European materials development: Results and perspective. Fusion Engineering and Design, 2019, 146, 1300-1307.	1.9	50
50	Master Curve Fracture Toughness Characterization of Eurofer97 Steel Variants Using Miniature Multi-Notch Bend Bar Specimens for Fusion Applications. , 2019, , .		3
51	Tensile properties of baseline and advanced tungsten grades for fusion applications. International Journal of Refractory Metals and Hard Materials, 2018, 75, 153-162.	3.8	61
52	Radiation damage studies in fusion reactor steels by means of small-angle neutron scattering (SANS). Physica B: Condensed Matter, 2018, 551, 407-412.	2.7	6
53	TEM characterization on new 9% Cr advanced steels thermomechanical treated after tempering. Journal of Nuclear Materials, 2018, 500, 1-10.	2.7	19
54	Micro-structural effects of irradiation temperature and helium content in neutron irradiated B-alloyed Eurofer97-1 steel. Nuclear Materials and Energy, 2018, 17, 40-47.	1.3	7

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55	Interfacial characterization by TEM and nanoindentation of W-Eurofer brazed joints for the first wall component of the DEMO fusion reactor. Materials Characterization, 2018, 142, 162-169.	4.4	18
56	Investigation of precipitate in an austenitic ODS steel containing a carbon-rich process control agent. Nuclear Materials and Energy, 2018, 15, 237-243.	1.3	13
57	Improvement of EUROFER's mechanical properties by optimized chemical compositions and thermo-mechanical treatments. Nuclear Materials and Energy, 2018, 16, 88-94.	1.3	16
58	Processing of complex near-net-shaped tungsten parts by PIM. Nuclear Materials and Energy, 2018, 16, 71-75.	1.3	11
59	Development of EUROFER97 database and material property handbook. Fusion Engineering and Design, 2018, 135, 9-14.	1.9	35
60	A review of impact properties of tungsten materials. Fusion Engineering and Design, 2018, 135, 196-203.	1.9	30
61	Master Curve Fracture Toughness Characterization of Eurofer97 Using Miniature Multi-Notch Bend Bar Specimens for Fusion Applications. , 2018, , .		3
62	3D Structural Analysis of Selected High-Temperature Materials. Praktische Metallographie/Practical Metallography, 2018, 55, 424-446.	0.3	0
63	Measurements and controls implementation for WEST. Fusion Engineering and Design, 2017, 123, 1029-1032.	1.9	8
64	Production, microstructure and mechanical properties of two different austenitic ODS steels. Journal of Nuclear Materials, 2017, 487, 348-361.	2.7	30
65	Development of advanced high heat flux and plasma-facing materials. Nuclear Fusion, 2017, 57, 092007.	3.5	189
66	Development of next generation tempered and ODS reduced activation ferritic/martensitic steels for fusion energy applications. Nuclear Fusion, 2017, 57, 092005.	3.5	177
67	Choice of a low operating temperature for the DEMO EUROFER97 divertor cassette. Fusion Engineering and Design, 2017, 124, 655-658.	1.9	32
68	Advanced materials for a damage resilient divertor concept for DEMO: Powder-metallurgical tungsten-fibre reinforced tungsten. Fusion Engineering and Design, 2017, 124, 964-968.	1.9	40
69	Ductilisation of tungsten (W): Tungsten laminated composites. International Journal of Refractory Metals and Hard Materials, 2017, 69, 66-109.	3.8	57
70	Assessment of industrial nitriding processes for fusion steel applications. Nuclear Materials and Energy, 2017, 13, 90-98.	1.3	1
71	Ductilisation of tungsten (W): On the increase of strength AND room-temperature tensile ductility through cold-rolling. International Journal of Refractory Metals and Hard Materials, 2017, 64, 261-278.	3.8	52
72	Effect of neutron irradiation on the microstructure of tungsten. Nuclear Materials and Energy, 2016, 9, 480-483.	1.3	64

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73	Materials for DEMO and reactor applicationsâ€"boundary conditions and new concepts. Physica Scripta, 2016, T167, 014002.	2.5	85
74	Ductilisation of tungsten (W): On the shift of the brittle-to-ductile transition (BDT) to lower temperatures through cold rolling. International Journal of Refractory Metals and Hard Materials, 2016, 54, 351-369.	3.8	97
75	Recent status and improvement of reduced-activation ferritic-martensitic steels for high-temperature service. Journal of Nuclear Materials, 2016, 479, 515-523.	2.7	87
76	Irradiation effects in tungsten-copper laminate composite. Journal of Nuclear Materials, 2016, 481, 134-146.	2.7	27
77	Conceptual design studies for the European DEMO divertor: Rationale and first results. Fusion Engineering and Design, 2016, 109-111, 1598-1603.	1.9	108
78	Progress in the engineering design and assessment of the European DEMO first wall and divertor plasma facing components. Fusion Engineering and Design, 2016, 109-111, 917-924.	1.9	74
79	European DEMO divertor target: Operational requirements and material-design interface. Nuclear Materials and Energy, 2016, 9, 171-176.	1.3	119
80	Numerical exploration into the potential of tungsten reinforced CuCrZr matrix composites. Journal of Nuclear Materials, 2016, 470, 13-29.	2.7	11
81	Ductilisation of tungsten (W) through cold-rolling: R-curve behaviour. International Journal of Refractory Metals and Hard Materials, 2016, 58, 22-33.	3.8	40
82	Manufacturing and characterization of PIM-W materials as plasma facing materials. Physica Scripta, 2016, T167, 014056.	2.5	4
83	Improvement of reduced activation 9%Cr steels by ausforming. Nuclear Materials and Energy, 2016, 6, 12-17.	1.3	43
84	Overview of the design approach and prioritization of R&D activities towards an EU DEMO. Fusion Engineering and Design, 2016, 109-111, 1464-1474.	1.9	178
85	Neutron diffraction stress determination in W-laminates for structural divertor applications. Nuclear Materials and Energy, 2015, 3-4, 37-42.	1.3	2
86	Tungsten (W) Laminate Pipes for Innovative High Temperature Energy Conversion Systems. Advanced Engineering Materials, 2015, 17, 491-501.	3.5	32
87	Optimization of growth parameters for growth of high quality heteroepitaxial 3Câ€"SiC films at 1200Â°C. Thin Solid Films, 2015, 577, 88-93.	1.8	12
88	Microstructural anisotropy of ferritic ODS alloys after different production routes. Fusion Engineering and Design, 2015, 98-99, 1986-1990.	1.9	13
89	Tungsten laminates made of ultrafine-grained (UFG) tungsten foil â€" Ageing of tungstenâ€"titanium (Wâ€"Ti) laminates. International Journal of Refractory Metals and Hard Materials, 2015, 51, 264-274.	3.8	16
90	Testing candidate interlayers for an enhanced water-cooled divertor target. Fusion Engineering and Design, 2015, 98-99, 1323-1327.	1.9	1

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91	Mechanical and microstructural investigations of tungsten and doped tungsten materials produced via powder injection molding. Nuclear Materials and Energy, 2015, 3-4, 22-31.	1.3	60
92	Enhancing the DEMO divertor target by interlayer engineering. Fusion Engineering and Design, 2015, 98-99, 1216-1220.	1.9	28
93	The nature of the brittle-to-ductile transition of ultra fine grained tungsten (W) foil. International Journal of Refractory Metals and Hard Materials, 2015, 50, 9-15.	3.8	51
94	Microstructural and mechanical characterization of annealed tungsten (W) and potassium-doped tungsten foils. International Journal of Refractory Metals and Hard Materials, 2015, 48, 145-149.	3.8	27
95	Materials R&D for a timely DEMO: Key findings and recommendations of the EU Roadmap Materials Assessment Group. Fusion Engineering and Design, 2014, 89, 1586-1594.	1.9	120
96	Developing structural, high-heat flux and plasma facing materials for a near-term DEMO fusion power plant: The EU assessment. Journal of Nuclear Materials, 2014, 455, 277-291.	2.7	210
97	Effect of helium implantation on mechanical properties of EUROFER97 evaluated by nanoindentation. Journal of Nuclear Materials, 2014, 448, 301-309.	2.7	16
98	Overview of the Structural Materials Program for Fusion Reactors under EFDA. Fusion Science and Technology, 2014, 66, 38-45.	1.1	7
99	Creep-Fatigue Interaction in Eurofer 3 Electron Beam Welds. Fusion Science and Technology, 2014, 66, 131-135.	1.1	1
100	Investigation of European tungsten materials exposed to high heat flux H/He neutral beams. Journal of Nuclear Materials, 2013, 442, S256-S260.	2.7	21
101	Tungsten foil laminate for structural divertor applications â€“ Joining of tungsten foils. Journal of Nuclear Materials, 2013, 436, 47-55.	2.7	30
102	A brief summary of the progress on the EFDA tungsten materials program. Journal of Nuclear Materials, 2013, 442, S173-S180.	2.7	69
103	Recent progress in R&D on tungsten alloys for divertor structural and plasma facing materials. Journal of Nuclear Materials, 2013, 442, S181-S189.	2.7	272
104	Assessment of copper based materials for the Water-Cooled Divertor concept of the DEMO European Fusion reactor. , 2013, , .		1
105	Characterization of ODS (Oxide Dispersion Strengthened) Eurofer/Eurofer dissimilar electron beam welds. Journal of Nuclear Materials, 2013, 442, S552-S556.	2.7	12
106	Microstructure and mechanical properties of a Wâ€“2wt.%Y2O3 composite produced by sintering and hot forging. Journal of Nuclear Materials, 2013, 442, S225-S228.	2.7	43
107	Investigation on different oxides as candidates for nano-sized ODS particles in reduced-activation ferritic (RAF) steels. Journal of Nuclear Materials, 2013, 442, 444-448.	2.7	45
108	A failâ€“safe and cost effective fabrication route for blanket First Walls. Journal of Nuclear Materials, 2013, 442, 538-541.	2.7	3

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109	Investigations of dissimilar welds of the high temperature steels P91 and PM2000. Fusion Engineering and Design, 2013, 88, 2539-2542.	1.9	9
110	KATHELO: A new high heat flux component testing facility. Fusion Engineering and Design, 2013, 88, 854-857.	1.9	7
111	Charpy impact properties of pure tungsten plate material in as-received and recrystallized condition (1h at 2000Â°C (2273K)). Journal of Nuclear Materials, 2013, 442, S204-S207.	2.7	38
112	Tungsten foil laminate for structural divertor applications â€“ Tensile test properties of tungsten foil. Journal of Nuclear Materials, 2013, 434, 357-366.	2.7	51
113	Recent progress in research on tungsten materials for nuclear fusion applications in Europe. Journal of Nuclear Materials, 2013, 432, 482-500.	2.7	610
114	The Impact of Refractory Material Properties on the Helium Cooled Divertor Design. Fusion Science and Technology, 2012, 61, 381-384.	1.1	28
115	Technology Developments at KIT Towards a Magnetic Confinement Fusion Power Plant. Fusion Science and Technology, 2012, 61, 64-69.	1.1	5
116	Optimization and limitations of known DEMO divertor concepts. Fusion Engineering and Design, 2012, 87, 718-721.	1.9	36
117	TEM study of mechanically alloyed ODS steel powder. Journal of Nuclear Materials, 2012, 428, 165-169.	2.7	12
118	Tungsten foil laminate for structural divertor applications â€“ Basics and outlook. Journal of Nuclear Materials, 2012, 423, 1-8.	2.7	79
119	Tungsten foil laminate for structural divertor applications â€“ Analyses and characterisation of tungsten foil. Journal of Nuclear Materials, 2012, 424, 197-203.	2.7	63
120	Deep drawing of tungsten plates for structural divertor applications in future fusion devices. Fusion Engineering and Design, 2011, 86, 2949-2953.	1.9	11
121	W-Based Alloys for Advanced Divertor Designs: Options and Environmental Impact of State-of-the-Art Alloys. Fusion Science and Technology, 2011, 60, 185-189.	1.1	27
122	Innovative materials for Gen IV systems and transmutation facilities: The cross-cutting research project GETMAT. Nuclear Engineering and Design, 2011, 241, 3514-3520.	1.7	33
123	Review on the EFDA work programme on nano-structured ODS RAF steels. Journal of Nuclear Materials, 2011, 417, 149-153.	2.7	66
124	Review of candidate welding processes of RAFM steels for ITER test blanket modules and DEMO. Journal of Nuclear Materials, 2011, 417, 43-50.	2.7	64
125	Development of welding technologies for the manufacturing of European Tritium Breeder blanket modules. Journal of Nuclear Materials, 2011, 417, 36-42.	2.7	29
126	Creepâ€“fatigue interaction and related structure property correlations of EUROFER97 steel at 550Â°C by decoupling creep and fatigue load. Journal of Nuclear Materials, 2011, 417, 16-19.	2.7	13

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127	Modelling structural and plasma facing materials for fusion power plants: Recent advances and outstanding issues in the EURATOM fusion materials programme. Journal of Nuclear Materials, 2011, 417, 1042-1049.	2.7	8
128	Review on the EFDA programme on tungsten materials technology and science. Journal of Nuclear Materials, 2011, 417, 463-467.	2.7	157
129	Feâ€“Crâ€“V ferritic steels for possible nuclear applications. Journal of Nuclear Materials, 2011, 409, 140-146.	2.7	6
130	Low activation steels welding with PWHT and coating for ITER test blanket modules and DEMO. Journal of Nuclear Materials, 2011, 409, 156-162.	2.7	18
131	Influence of microstructure and notch fabrication on impact bending properties of tungsten materials. International Journal of Refractory Metals and Hard Materials, 2010, 28, 679-686.	3.8	88
132	High heat flux componentsâ€”Readiness to proceed from near term fusion systems to power plants. Fusion Engineering and Design, 2010, 85, 93-108.	1.9	113
133	Specific welds for test blanket modules. Journal of Nuclear Materials, 2009, 386-388, 471-474.	2.7	27
134	Diffusion weld study for Test Blanket Module fabrication. Fusion Engineering and Design, 2009, 84, 1602-1605.	1.9	15
135	Transmutation and activation analysis for divertor materials in a HCLL-type fusion power reactor. Journal of Nuclear Materials, 2009, 386-388, 789-792.	2.7	11
136	European cross-cutting research on structural materials for Generation IV and transmutation systems. Journal of Nuclear Materials, 2009, 392, 316-323.	2.7	59
137	He-Cooled Divertor Development Towards DEMO. Fusion Science and Technology, 2009, 56, 1013-1017.	1.1	11
138	Fracture Behaviour of Tungsten Based Alloys Depending on Microstructure and Notch Fabrication Method. Fusion Science and Technology, 2009, 56, 1018-1022.	1.1	16
139	Formation and growth of complex precipitates in 316L austenitic steel during long-term annealing experiments. Journal of Materials Science, 2008, 43, 2541-2549.	3.7	3
140	Fissile core and Tritium-Breeding Blanket: structural materials and their requirements. Comptes Rendus Physique, 2008, 9, 287-302.	0.9	70
141	Impact Bending Tests on Selected Refractory Materials. Advanced Materials Research, 2008, 59, 101-104.	0.3	20
142	Modelling irradiation effects in fusion materials. Fusion Engineering and Design, 2007, 82, 2413-2421.	1.9	40
143	Precipitation in AISI 316L(N) during creep tests at 550 and 600Â°C up to 10 years. Journal of Nuclear Materials, 2007, 362, 132-138.	2.7	84
144	A comprising steady-state creep model for the austenitic AISI 316 L(N) steel. Journal of Nuclear Materials, 2007, 367-370, 915-919.	2.7	22

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145	Development of a helium-cooled divertor: Material choice and technological studies. Journal of Nuclear Materials, 2007, 367-370, 1416-1421.	2.7	146
146	Displacement cascades in Feâ€“Cr: A molecular dynamics study. Journal of Nuclear Materials, 2006, 349, 119-132.	2.7	110
147	The European effort towards the development of a demo structural material: Irradiation behaviour of the European reference RAFM steel EUROFER. Fusion Engineering and Design, 2006, 81, 917-923.	1.9	42
148	Limitations of W and Wâ€“1%La2O3 for use as structural materials. Journal of Nuclear Materials, 2005, 342, 20-25.	2.7	105
149	Creep strength of reduced activation ferritic/martensitic steel Euroferâ€™97. Fusion Engineering and Design, 2005, 75-79, 1003-1008.	1.9	32
150	Present development status of EUROFER and ODS-EUROFER for application in blanket concepts. Fusion Engineering and Design, 2005, 75-79, 989-996.	1.9	412
151	Towards reduced activation structural materials data for fusion DEMO reactors. Nuclear Fusion, 2005, 45, 649-655.	3.5	66
152	Electron in an Interaction Potential of General Shape. Journal of Computational and Theoretical Nanoscience, 2005, 2, 362-369.	0.4	0
153	Self-organizing processes in connection with metastable nanocluster states. Journal of Vacuum Science and Technology A: Vacuum, Surfaces and Films, 2004, 22, 1433-1438.	2.1	0
154	Development of a helium-cooled divertor concept: design-related requirements on materials and fabrication technology. Journal of Nuclear Materials, 2004, 329-333, 1594-1598.	2.7	120
155	Welcome to the <I>Journal of Computational and Theoretical Nanoscience</I>. Journal of Computational and Theoretical Nanoscience, 2004, 1, 1-2.	0.4	3
156	Metallic Nanoclusters: Computational Investigations of their Applicability as Building Blocks in Nanotechnology. Journal of Computational and Theoretical Nanoscience, 2004, 1, 40-46.	0.4	8
157	Special Issues, Special Sections, and Papers on Basic Physics. Journal of Computational and Theoretical Nanoscience, 2004, 1, 341-342.	0.4	1
158	Stability of an exciton bound to an ionized donor in quantum dots. Physics Letters, Section A: General, Atomic and Solid State Physics, 2003, 308, 219-225.	2.1	40
159	NOVEL NUMERICAL METHOD FOR THE SOLUTION OF SCHRÃ–DINGER'S EQUATION: EXCITON ENERGY OF CdS QUANTUM DOTS. International Journal of Modern Physics B, 2002, 16, 4093-4103.	2.0	4
160	Computational Engineering of Metallic Nanostructures and Nanomachines. Journal of Nanoscience and Nanotechnology, 2002, 2, 679-685.	0.9	2
161	Can Biological Effects Emerge from Inorganic Nano-Systems?. , 2002, , 179-197.		0
162	EXACT NUMERICAL SOLUTION OF SCHRÃ–DINGER'S EQUATION FOR A PARTICLE IN AN INTERACTION POTENTIAL OF GENERAL SHAPE. International Journal of Modern Physics B, 2002, 16, 4081-4092.	2.0	46

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163	Computational Engineering of Metallic Nanostructures and Nanomachines. Journal of Nanoscience and Nanotechnology, 2002, 2, 679-685.	0.9	1
164	Dissipative dynamics of a parabolic confined particle in the presence of magnetic field. Physica A: Statistical Mechanics and Its Applications, 2001, 292, 238-254.	2.6	2
165	Mechanical behavior of reduced-activation and conventional martensitic steels after neutron irradiation in the range 250â€“450Â°C. Journal of Nuclear Materials, 2000, 283-287, 353-357.	2.7	35
166	V-alloy embrittlement by irradiation in a cooling gas environment. Journal of Nuclear Materials, 2000, 283-287, 498-502.	2.7	3
167	THERMAL STABILITY AND SPECIFIC MATERIAL PROPERTIES OF NANOSYSTEMS. Modern Physics Letters B, 2000, 14, 621-629.	1.9	5
168	Mechanical Properties and Microstructure of HFR-Irradiated Ferritic/Martensitic Low-Activation Alloys. , 2000, , 597-611.		8
169	The effect of tantalum on the mechanical properties of a 9Crâ€“2Wâ€“0.25Vâ€“0.07Taâ€“0.1C steel. Journal of Nuclear Materials, 1999, 273, 146-154.	2.7	53
170	Influence of helium on impact properties of reduced-activation ferritic/martensitic Cr-steels. Journal of Nuclear Materials, 1999, 271-272, 450-454.	2.7	45
171	Embrittlement behaviour of different international low activation alloys after neutron irradiation. Journal of Nuclear Materials, 1998, 258-263, 1147-1152.	2.7	76
172	Structure and phonon density of states in nanoclusters: Molecular dynamics study for Al. Journal of Vacuum Science & Technology an Official Journal of the American Vacuum Society B, Microelectronics Processing and Phenomena, 1997, 15, 1610.	1.6	3
173	Electronic States of Nano-Systems. International Journal of Modern Physics B, 1997, 11, 767-777.	2.0	1
174	Effect of ageing and specimen size on the impact properties of MANET II steel. International Journal of Pressure Vessels and Piping, 1997, 74, 39-47.	2.6	7
175	Charpy impact properties of low activation alloys for fusion applications after neutron irradiation. Journal of Nuclear Materials, 1996, 233-237, 351-355.	2.7	20
176	Low temperature embrittlement behaviour of different ferritic-martensitic alloys for fusion applications. Journal of Nuclear Materials, 1996, 233-237, 229-232.	2.7	9
177	Charpy impact properties of martensitic 10.6% Cr steel (MANET-I) before and after neutron exposure. Fusion Engineering and Design, 1995, 29, 365-370.	1.9	21
178	State of the art: development of a helium-cooled divertor for demo. , 0, , .		5
179	Tungsten as a Structural Divertor Material. Advances in Science and Technology, 0, , .	0.2	60
180	First-Principles Modeling of Tungsten-Based Alloys for Fusion Power Plant Applications. Key Engineering Materials, 0, 465, 15-20.	0.4	16