

Gregory De Temmerman

List of Publications by Year in descending order

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194
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#	ARTICLE	IF	CITATIONS
1	Operating a full tungsten actively cooled tokamak: overview of WEST first phase of operation. Nuclear Fusion, 2022, 62, 042007.	3.5	39
2	Investigation of plasma wall interactions between tungsten plasma facing components and helium plasmas in the WEST tokamak. Nuclear Fusion, 2022, 62, 076028.	3.5	22
3	Identifying microstructural changes responsible for retarded grain growth during tungsten recrystallization after helium plasma exposure. Journal of Nuclear Materials, 2022, 559, 153448.	2.7	4
4	Overview of JET results for optimising ITER operation. Nuclear Fusion, 2022, 62, 042026.	3.5	52
5	Improved scaling law for the prediction of deuterium retention in beryllium co-deposits. Nuclear Fusion, 2022, 62, 036006.	3.5	4
6	First plasma exposure of a pre-damaged ITER-like plasma-facing unit in the WEST tokamak: procedure for the PFU preparation and lessons learned. Nuclear Fusion, 2022, 62, 056010.	3.5	5
7	Influence of interface conditions on hydrogen transport studies. Nuclear Fusion, 2021, 61, 036038.	3.5	8
8	First mirror erosion-deposition studies in JET using an ITER-like mirror test assembly. Nuclear Fusion, 2021, 61, 046022.	3.5	13
9	Recrystallization-mediated crack initiation in tungsten under simultaneous high-flux hydrogen plasma loads and high-cycle transient heating. Nuclear Fusion, 2021, 61, 046018.	3.5	14
10	RF discharge mirror cleaning system development for ITER diagnostics. Fusion Engineering and Design, 2021, 164, 112162.	1.9	16
11	Data on erosion and hydrogen fuel retention in Beryllium plasma-facing materials. Nuclear Materials and Energy, 2021, 27, 100994.	1.3	21
12	Power deposition behavior of high-density transient hydrogen plasma on tungsten in Magnum-PSI. Plasma Physics and Controlled Fusion, 2021, 63, 085016.	2.1	4
13	Very high-resolution infrared imagery of misaligned tungsten monoblock edge heating in the WEST tokamak. Nuclear Materials and Energy, 2021, 27, 100910.	1.3	4
14	Combined high fluence and high cycle number transient loading of ITER-like monoblocks in Magnum-PSI. Nuclear Fusion, 2021, 61, 116045.	3.5	9
15	The influence of D_2 pressure on D retention and release from Be co-deposits. Nuclear Materials and Energy, 2021, 28, 101023.	1.3	1
16	Fuel retention in WEST and ITER divertors based on FESTIM monoblock simulations. Nuclear Fusion, 2021, 61, 126001.	3.5	9
17	Nitrogen retention and ammonia production on tungsten. Nuclear Fusion, 2021, 61, 126067.	3.5	4
18	Plasma-activated catalytic formation of ammonia from N_2 and H_2 : influence of temperature and noble gas addition. Nuclear Fusion, 2020, 60, 016026.	3.5	13

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19	Recrystallization at high temperature of two tungsten materials complying with the ITER specifications. Journal of Nuclear Materials, 2020, 542, 152418.	2.7	25
20	Parametric study of hydrogenic inventory in the ITER divertor based on machine learning. Scientific Reports, 2020, 10, 17798.	3.3	11
21	Sticking Probability of Ammonia Molecules on Tungsten and 316L Stainless Steel Surfaces. Journal of Physical Chemistry C, 2020, 124, 17566-17577.	3.1	7
22	Three mechanisms of hydrogen-induced dislocation pinning in tungsten. Nuclear Fusion, 2020, 60, 086015.	3.5	12
23	Helium irradiation effects on the surface modification and recrystallization of tungsten. Physica Scripta, 2020, T171, 014050.	2.5	2
24	Scanning electron microscopy analyses of an ITER plasma-facing unit mockup exposed to extreme ion fluences in Magnum-PSI. Physica Scripta, 2020, T171, 014026.	2.5	8
25	Deuterium retention in Be-D co-deposits formed over an ITER relevant parameter space. Physica Scripta, 2020, T171, 014014.	2.5	11
26	CRDS modelling of deuterium release from co-deposited beryllium layers in temperature programmed and laser induced desorption experiments. Physica Scripta, 2020, T171, 014053.	2.5	5
27	ITER monoblock performance under lifetime loading conditions in Magnum-PSI. Physica Scripta, 2020, T171, 014065.	2.5	22
28	Effect of helium ion irradiation on tungsten recrystallization. Physica Scripta, 2020, T171, 014004.	2.5	7
29	Fracture behavior of tungsten-based composites exposed to steady-state/transient hydrogen plasma. Nuclear Fusion, 2020, 60, 046029.	3.5	13
30	Multidimensional finite-element simulations of the diffusion and trapping of hydrogen in plasma-facing components including thermal expansion. Physica Scripta, 2020, T171, 014011.	2.5	12
31	Impact of H-mode plasma operation on pre-damaged tungsten divertor tiles in ASDEX Upgrade. Physica Scripta, 2020, T171, 014037.	2.5	4
32	Efficiency of laser-induced desorption of D from Be/D layers and surface modifications due to LID. Physica Scripta, 2020, T171, 014075.	2.5	11
33	First analysis of the misaligned leading edges of ITER-like plasma facing units using a very high resolution infrared camera in WEST. Nuclear Fusion, 2020, 60, 106020.	3.5	18
34	Blister-dominated retention mechanism in tungsten exposed to high-fluence deuterium plasma. Nuclear Fusion, 2020, 60, 126034.	3.5	20
35	Long discharges in a steady state with D_{2} and N_{2} on the actively cooled tungsten upper divertor in WEST. Nuclear Fusion, 2020, 60, 126046.	3.5	9
36	Deuterium removal from beryllium co-deposits by simulated strike-point sweeping. Nuclear Materials and Energy, 2020, 24, 100750.	1.3	3

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37	Exposure of Indian RAFM under variation of He ⁺ flux and target temperature in the CIMPLE-PSI linear device. Nuclear Fusion, 2020, 60, 106026.	3.5	2
38	TMAP modeling of D release from baked multi-layer Be-D co-deposits. Physica Scripta, 2020, T171, 014043.	2.5	2
39	Plasma-assisted catalytic formation of ammonia in N ₂ -H ₂ plasma on a tungsten surface. Physical Chemistry Chemical Physics, 2019, 21, 16623-16633.	2.8	31
40	Physics basis for the first ITER tungsten divertor. Nuclear Materials and Energy, 2019, 20, 100696.	1.3	307
41	Temperature dependence of retarded recrystallisation in helium plasma-exposed tungsten. Nuclear Fusion, 2019, 59, 096031.	3.5	10
42	Design, development and recent experiments of the CIMPLE-PSI device. Nuclear Fusion, 2019, 59, 112008.	3.5	5
43	WallDYN simulations of material migration and fuel retention in ITER low power H plasmas and high power neon-seeded DT plasmas. Nuclear Materials and Energy, 2019, 20, 100674.	1.3	14
44	Accumulation of beryllium dust in ITER diagnostic ports after off-normal events. Nuclear Materials and Energy, 2019, 20, 100684.	1.3	4
45	Laser-Induced Desorption of co-deposited Deuterium in Beryllium Layers on Tungsten. Nuclear Materials and Energy, 2019, 19, 503-509.	1.3	15
46	Evolution of nitrogen concentration and ammonia production in N ₂ -seeded H-mode discharges at ASDEX Upgrade. Nuclear Fusion, 2019, 59, 046010.	3.5	22
47	A growth/annealing equilibrium model for helium-induced nanostructure with application to ITER. Nuclear Materials and Energy, 2019, 19, 255-261.	1.3	37
48	Numerical simulation by finite element modelling of diffusion and transient hydrogen trapping processes in plasma facing components. Nuclear Materials and Energy, 2019, 19, 42-46.	1.3	15
49	Studies on synthesis of plasma fusion relevant tungsten dust particles and measurement of their hydrogen absorption properties. Fusion Engineering and Design, 2018, 127, 120-126.	1.9	6
50	Design optimization of the ITER tungsten divertor vertical targets. Fusion Engineering and Design, 2018, 127, 66-72.	1.9	55
51	Suppression of deuterium-induced blistering in pre-damaged tungsten exposed to short-duration deuterium plasma. Journal of Nuclear Materials, 2018, 500, 295-300.	2.7	18
52	The influence of plasma-surface interaction on the performance of tungsten at the ITER divertor vertical targets. Plasma Physics and Controlled Fusion, 2018, 60, 044018.	2.1	149
53	Quartz micro-balance and in situ XPS study of the adsorption and decomposition of ammonia on gold, tungsten, boron, beryllium and stainless steel surfaces. Nuclear Fusion, 2018, 58, 106012.	3.5	10
54	Retarded recrystallization of helium-exposed tungsten. Nuclear Fusion, 2018, 58, 106011.	3.5	31

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55	Survival and in-vessel redistribution of beryllium droplets after ITER disruptions. Nuclear Fusion, 2018, 58, 076008.	3.5	23
56	Tungsten dust remobilization under steady-state and transient plasma conditions. Nuclear Materials and Energy, 2017, 12, 569-574.	1.3	20
57	Surface modification of He pre-exposed tungsten samples by He plasma impact in the divertor manipulator of ASDEX Upgrade. Nuclear Materials and Energy, 2017, 12, 575-581.	1.3	18
58	Engineering design and analysis of an ITER-like first mirror test assembly on JET. Fusion Engineering and Design, 2017, 123, 1054-1057.	1.9	5
59	Strong sub-surface plastic deformation induced by high flux plasma in tungsten. Fusion Engineering and Design, 2017, 124, 405-409.	1.9	14
60	Mitigated blistering and deuterium retention in tungsten exposed to high-flux deuterium-neon mixed plasmas. Nuclear Fusion, 2017, 57, 046028.	3.5	26
61	Plasma pressure and particle loss studies in the Pilot-PSI high flux linear plasma generator. Nuclear Materials and Energy, 2017, 12, 1088-1093.	1.3	6
62	Beryllium layer response to ITER-like ELM plasma pulses in QSPA-Be. Nuclear Materials and Energy, 2017, 12, 433-440.	1.3	9
63	Plasma-wall interaction studies in the full-W ASDEX upgrade during helium plasma discharges. Nuclear Fusion, 2017, 57, 066015.	3.5	16
64	Effects of stress-relief pre-annealing on deuterium trapping and diffusion in tungsten. Fusion Engineering and Design, 2017, 125, 526-530.	1.9	5
65	Remobilization of tungsten dust from castellated plasma-facing components. Nuclear Materials and Energy, 2017, 12, 536-540.	1.3	8
66	Physics conclusions in support of ITER W divertor monoblock shaping. Nuclear Materials and Energy, 2017, 12, 60-74.	1.3	128
67	Efficiency of thermal outgassing for tritium retention measurement and removal in ITER. Nuclear Materials and Energy, 2017, 12, 267-272.	1.3	63
68	Long-term fuel retention and release in JET ITER-Like Wall at ITER-relevant baking temperatures. Nuclear Fusion, 2017, 57, 086024.	3.5	25
69	Overview of progress in European medium sized tokamaks towards an integrated plasma-edge/wall solution. Nuclear Fusion, 2017, 57, 102014.	3.5	23
70	Plasma expansion synthesis of tungsten nanopowder. Journal of Alloys and Compounds, 2017, 725, 606-615.	5.5	19
71	Power deposition on misaligned castellated tungsten blocks in the Magnum-PSI and Pilot-PSI linear devices. Nuclear Fusion, 2017, 57, 126025.	3.5	13
72	Mechanical properties of tungsten following rhenium ion and helium plasma exposure. Nuclear Materials and Energy, 2017, 12, 1336-1341.	1.3	13

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73	Surface modifications and deuterium retention in polycrystalline and single crystal tungsten as a function of particle flux and temperature. <i>Journal of Nuclear Materials</i> , 2017, 495, 211-219.	2.7	18
74	Microstructure and nano-hardness of single crystal tungsten exposed to high flux deuterium plasma. <i>Physica Scripta</i> , 2017, T170, 014064.	2.5	4
75	Characterization and origin of large size dust particles produced in the Alcator C-Mod tokamak. <i>Nuclear Materials and Energy</i> , 2017, 11, 12-19.	1.3	20
76	ELM-induced arcing on tungsten fuzz in the COMPASS divertor region. <i>Journal of Nuclear Materials</i> , 2017, 492, 204-212.	2.7	14
77	Sub-surface microstructure of single and polycrystalline tungsten after high flux plasma exposure studied by TEM. <i>Applied Surface Science</i> , 2017, 393, 330-339.	6.1	20
78	Response of tungsten surfaces to helium and hydrogen plasma exposure under ITER relevant steady state and repetitive transient conditions. <i>Nuclear Fusion</i> , 2017, 57, 126009.	3.5	35
79	Investigation of He-W interactions using DiMES on DIII-D. <i>Physica Scripta</i> , 2016, T167, 014054.	2.5	19
80	Dislocation-mediated trapping of deuterium in tungsten under high-flux high-temperature exposures. <i>Journal of Nuclear Materials</i> , 2016, 479, 307-315.	2.7	13
81	Tailoring the charged particle fluxes across the target surface of Magnum-PSI. <i>Plasma Sources Science and Technology</i> , 2016, 25, 025023.	3.1	14
82	Collective Thomson scattering system for determination of ion properties in a high flux plasma beam. <i>Applied Physics Letters</i> , 2016, 109, .	3.3	16
83	Effect of rhenium addition on tungsten fuzz formation in helium plasmas. <i>Journal of Nuclear Materials</i> , 2016, 474, 99-104.	2.7	28
84	Interaction of adhered metallic dust with transient plasma heat loads. <i>Nuclear Fusion</i> , 2016, 56, 066010.	3.5	27
85	Mechanism for orientation dependence of blisters on W surface exposed to D plasma at low temperature. <i>Journal of Nuclear Materials</i> , 2016, 477, 165-171.	2.7	44
86	Development of a plasma assisted ITER level controlled heat source and observation of novel micro/nanostructures produced upon exposure of tungsten targets. <i>Fusion Engineering and Design</i> , 2016, 106, 63-70.	1.9	3
87	Deuterium blistering in tungsten and tungsten vanadium alloys. <i>Fusion Engineering and Design</i> , 2016, 107, 25-31.	1.9	15
88	Investigation of arcing on fiber-formed nanostructured tungsten by pulsed plasma during steady state plasma irradiation. <i>Fusion Engineering and Design</i> , 2016, 112, 156-161.	1.9	21
89	Observation of a helium ion energy threshold for retention in tungsten exposed to hydrogen/helium mixture plasma. <i>Nuclear Fusion</i> , 2016, 56, 104002.	3.5	19
90	Blistering on tungsten surface exposed to high flux deuterium plasma. <i>Journal of Nuclear Materials</i> , 2016, 471, 51-58.	2.7	31

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91	Surface Modifications Induced by High Fluxes of Low Energy Helium Ions. Scientific Reports, 2015, 5, 9779.	3.3	39
92	Highly resolved measurements of dust motion in the sheath boundary of magnetized plasmas. Nuclear Fusion, 2015, 55, 112001.	3.5	25
93	Thermal shock behaviour of blisters on W surface during combined steady-state/pulsed plasma loading. Nuclear Fusion, 2015, 55, 113015.	3.5	6
94	Nanostructures and pinholes on W surfaces exposed to high flux D plasma at high temperatures. Journal of Nuclear Materials, 2015, 463, 312-315.	2.7	24
95	Effect of neon plasma pre-irradiation on surface morphology and deuterium retention of tungsten. Journal of Nuclear Materials, 2015, 463, 1025-1028.	2.7	25
96	Influence of tungsten microstructure and ion flux on deuterium plasma-induced surface modifications and deuterium retention. Journal of Nuclear Materials, 2015, 463, 320-324.	2.7	37
97	High flux irradiations of Li coatings on polycrystalline W and ATJ graphite with D, He, and He-seeded D plasmas at Magnum PSI. Journal of Nuclear Materials, 2015, 463, 1147-1151.	2.7	5
98	The effect of transient temporal pulse shape on surface temperature and tungsten damage. Nuclear Fusion, 2015, 55, 093027.	3.5	19
99	Impact of combined hydrogen plasma and transient heat loads on the performance of tungsten as plasma facing material. Nuclear Fusion, 2015, 55, 123017.	3.5	44
100	Impact of combined transient plasma/heat loads on tungsten performance below and above recrystallization temperature. Nuclear Fusion, 2015, 55, 123004.	3.5	45
101	Characterizing the recovery of a solid surface after tungsten nano-tendrils formation. Journal of Nuclear Materials, 2015, 463, 294-298.	2.7	11
102	Deuterium retention in tungsten exposed to mixed D + N plasma at divertor relevant fluxes in Magnum-PSI. Journal of Nuclear Materials, 2015, 463, 974-978.	2.7	13
103	Cross-section analysis of the Magnum-PSI plasma beam using a 2D multi-probe system. Plasma Sources Science and Technology, 2015, 24, 015014.	3.1	9
104	LIBS analysis of tungsten coatings exposed to Magnum PSI ELM-like plasma. Journal of Nuclear Materials, 2015, 463, 919-922.	2.7	25
105	Evolution of transiently melt damaged tungsten under ITER-relevant divertor plasma heat loading. Journal of Nuclear Materials, 2015, 463, 193-197.	2.7	16
106	Experimental investigation of neon seeding in the snowflake configuration in TCV. Journal of Nuclear Materials, 2015, 463, 1196-1199.	2.7	9
107	Modeling the reduction of gross lithium erosion observed under high-flux deuterium bombardment. Journal of Nuclear Materials, 2015, 463, 1169-1172.	2.7	12
108	Optimization of tungsten castellated structures for the ITER divertor. Journal of Nuclear Materials, 2015, 463, 174-179.	2.7	3

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109	Elastic-plastic adhesive impacts of tungsten dust with metal surfaces in plasma environments. Journal of Nuclear Materials, 2015, 463, 877-880.	2.7	19
110	Effect of high-flux H/He plasma exposure on tungsten damage due to transient heat loads. Journal of Nuclear Materials, 2015, 463, 198-201.	2.7	21
111	Early stage damage of ultrafine-grained tungsten materials exposed to low energy helium ion irradiation. Fusion Engineering and Design, 2015, 93, 9-14.	1.9	36
112	Deuterium-induced nanostructure formation on tungsten exposed to high-flux plasma. Journal of Nuclear Materials, 2015, 463, 308-311.	2.7	12
113	Plasma temperature rise toward the plasma-facing surface. Journal of Nuclear Materials, 2015, 463, 440-444.	2.7	3
114	Interaction of a tin-based capillary porous structure with ITER/DEMO relevant plasma conditions. Journal of Nuclear Materials, 2015, 463, 1256-1259.	2.7	27
115	Surface morphology changes and deuterium retention in Toughened, Fine-grained Recrystallized Tungsten under high-flux irradiation conditions. Journal of Nuclear Materials, 2015, 463, 1037-1040.	2.7	11
116	Surface morphology and deuterium retention in tungsten exposed to high flux D plasma at high temperatures. Journal of Nuclear Materials, 2015, 457, 213-219.	2.7	63
117	Synergy of plastic deformation and gas retention in tungsten. Nuclear Fusion, 2015, 55, 013007.	3.5	27
118	The occurrence and damage of unipolar arcing on fuzzy tungsten. Journal of Nuclear Materials, 2015, 463, 303-307.	2.7	27
119	Thermal response of nanostructured tungsten. Nuclear Fusion, 2014, 54, 033005.	3.5	66
120	Self-shielding of a plasma-exposed surface during extreme transient heat loads. Applied Physics Letters, 2014, 104, 124102.	3.3	10
121	Erosion of lithium coatings on TZM molybdenum and graphite during high-flux plasma bombardment. Fusion Engineering and Design, 2014, 89, 2857-2863.	1.9	22
122	Ultrafine tungsten as a plasma-facing component in fusion devices: effect of high flux, high fluence low energy helium irradiation. Nuclear Fusion, 2014, 54, 083013.	3.5	101
123	A high-repetition rate edge localised mode replication system for the Magnum-PSI and Pilot-PSI linear devices. Plasma Physics and Controlled Fusion, 2014, 56, 095004.	2.1	33
124	Research status and issues of tungsten plasma facing materials for ITER and beyond. Fusion Engineering and Design, 2014, 89, 901-906.	1.9	246
125	Spontaneous synthesis of carbon nanowalls, nanotubes and nanotips using high flux density plasmas. Carbon, 2014, 68, 695-707.	10.3	20
126	The effect of high-flux H plasma exposure with simultaneous transient heat loads on tungsten surface damage and power handling. Nuclear Fusion, 2014, 54, 123010.	3.5	49

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127	Nanostructuring of Iron Surfaces by Low-Energy Helium Ions. ACS Applied Materials & Interfaces, 2014, 6, 3462-3468.	8.0	40
128	Operational characteristics of the high flux plasma generator Magnum-PSI. Fusion Engineering and Design, 2014, 89, 2150-2154.	1.9	29
129	Study of deuterium retention on lithiated tungsten exposed to high-flux deuterium plasma using laser-induced breakdown spectroscopy. Fusion Engineering and Design, 2014, 89, 949-954.	1.9	23
130	Helium effects on tungsten surface morphology and deuterium retention. Journal of Nuclear Materials, 2013, 442, S267-S272.	2.7	83
131	Efficient Plasma Route to Nanostructure Materials: Case Study on the Use of m-WO ₃ for Solar Water Splitting. ACS Applied Materials & Interfaces, 2013, 5, 7621-7625.	8.0	96
132	Advanced divertor configurations with large flux expansion. Journal of Nuclear Materials, 2013, 438, S96-S101.	2.7	24
133	Performances of Rh and Mo mirrors under JET exposure. Journal of Nuclear Materials, 2013, 438, S1187-S1191.	2.7	14
134	Observations of orientation dependence of surface morphology in tungsten implanted by low energy and high flux D plasma. Journal of Nuclear Materials, 2013, 443, 452-457.	2.7	55
135	Overview of physics results from MAST towards ITER/DEMO and the MAST Upgrade. Nuclear Fusion, 2013, 53, 104008.	3.5	21
136	High heat flux capabilities of the Magnum-PSI linear plasma device. Fusion Engineering and Design, 2013, 88, 483-487.	1.9	103
137	Dust investigations in TEXTOR: Impact of dust on plasma-wall interactions and on plasma performance. Journal of Nuclear Materials, 2013, 438, S126-S132.	2.7	19
138	Helium effects on tungsten under fusion-relevant plasma loading conditions. Journal of Nuclear Materials, 2013, 438, S78-S83.	2.7	89
139	Carbon transport and escape fraction in a high density plasma beam. Journal of Nuclear Materials, 2013, 438, S629-S632.	2.7	5
140	Assessment of cleaning methods for first mirrors tested in JET for ITER. Journal of Nuclear Materials, 2013, 438, S1241-S1244.	2.7	6
141	Carbon migration during methane injection experiments under ITER divertor-relevant conditions. Journal of Nuclear Materials, 2013, 438, S686-S689.	2.7	2
142	Thermographic determination of the sheath heat transmission coefficient in a high density plasma. Journal of Nuclear Materials, 2013, 438, S431-S434.	2.7	12
143	Thermal shock behaviour of tungsten after high flux H-plasma loading. Journal of Nuclear Materials, 2013, 443, 497-501.	2.7	36
144	Erosion of marker coatings exposed to Pilot-PSI plasma. Journal of Nuclear Materials, 2013, 438, S754-S757.	2.7	7

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145	Helium concentration in tungsten nano-tendrils surface morphology using Elastic Recoil Detection. Journal of Nuclear Materials, 2013, 438, S913-S916.	2.7	30
146	Reduced damage threshold for tungsten using combined steady state and transient sources. Journal of Nuclear Materials, 2013, 438, S784-S787.	2.7	12
147	Comparison of tungsten nano-tendrils grown in Alcator C-Mod and linear plasma devices. Journal of Nuclear Materials, 2013, 438, S84-S89.	2.7	70
148	Synergistic effects of hydrogen plasma exposure, pulsed laser heating and temperature on rhodium surfaces. Journal of Nuclear Materials, 2013, 432, 388-394.	2.7	2
149	Power distribution in the snowflake divertor in TCV. Plasma Physics and Controlled Fusion, 2013, 55, 124027.	2.1	59
150	Laser-based diagnostics applications for plasma-surface interaction studies. Journal of Instrumentation, 2013, 8, C11011-C11011.	1.2	7
151	Chemical sputtering of graphite by low temperature nitrogen plasmas at various substrate temperatures and ion flux densities. Journal of Applied Physics, 2013, 114, .	2.5	10
152	Nanostructuring of molybdenum and tungsten surfaces by low-energy helium ions. Journal of Vacuum Science and Technology A: Vacuum, Surfaces and Films, 2012, 30, .	2.1	119
153	The target for the new plasma/wall experiment Magnum-PSI. Fusion Engineering and Design, 2011, 86, 1745-1748.	1.9	14
154	New linear plasma devices in the trilateral euregio cluster for an integrated approach to plasma surface interactions in fusion reactors. Fusion Engineering and Design, 2011, 86, 1797-1800.	1.9	36
155	Heat load asymmetries in MAST. Journal of Nuclear Materials, 2011, 415, S383-S386.	2.7	13
156	Diamond coatings exposure to fusion-relevant plasma conditions. Journal of Nuclear Materials, 2011, 415, S161-S164.	2.7	7
157	Production and characterization of transient heat and particle pulses in Pilot-PSI. Journal of Nuclear Materials, 2011, 415, S70-S73.	2.7	5
158	Strike-point splitting induced by external magnetic perturbations: Observations on JET and MAST and associated modelling. Journal of Nuclear Materials, 2011, 415, S914-S917.	2.7	48
159	Erosion yields of carbon under various plasma conditions in Pilot-PSI. Journal of Nuclear Materials, 2011, 415, S149-S152.	2.7	13
160	Removal of beryllium-containing films deposited in JET from mirror surfaces by laser cleaning. Journal of Nuclear Materials, 2011, 415, S1199-S1202.	2.7	30
161	Active control over carbon deposition on diagnostic components and in remote areas of ITER. Journal of Nuclear Materials, 2011, 417, 830-833.	2.7	5
162	Materials research under ITER-like divertor conditions at FOM Rijnhuizen. Journal of Nuclear Materials, 2011, 417, 457-462.	2.7	1

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163	ELM simulation experiments on Pilot-PSI using simultaneous high flux plasma and transient heat/particle source. Nuclear Fusion, 2011, 51, 073008.	3.5	82
164	Overview of the second stage in the comprehensive mirrors test in JET. Physica Scripta, 2011, T145, 014070.	2.5	20
165	Analyses of dust samples collected in the MAST tokamak. Journal of Nuclear Materials, 2010, 401, 130-137.	2.7	20
166	First Mirrors Test in JET for ITER: An overview of optical performance and surface morphology. Nuclear Instruments and Methods in Physics Research, Section A: Accelerators, Spectrometers, Detectors and Associated Equipment, 2010, 623, 818-822.	1.6	15
167	Density measurements using coherence imaging spectroscopy based on Stark broadening. Review of Scientific Instruments, 2010, 81, 10E521.	1.3	5
168	Dust creation and transport in MAST. Nuclear Fusion, 2010, 50, 105012.	3.5	44
169	Similarities and differences between dust produced in laboratory plasmas and in the MAST and Tore Supra tokamaks. Plasma Physics and Controlled Fusion, 2010, 52, 124007.	2.1	18
170	Surface analysis of CVD diamond exposed to fusion plasma. Diamond and Related Materials, 2010, 19, 818-823.	3.9	16
171	Revised scaling equation for the prediction of tritium retention in beryllium co-deposited layers. Nuclear Fusion, 2009, 49, 042002.	3.5	32
172	Edge localized mode control experiments on MAST using resonant magnetic perturbations from in-vessel coils. Plasma Physics and Controlled Fusion, 2009, 51, 124010.	2.1	39
173	Deuterium retention and release in tungsten co-deposited layers. Journal of Nuclear Materials, 2009, 389, 479-483.	2.7	55
174	Insight into the co-deposition of deuterium with beryllium: Influence of the deposition conditions on the deuterium retention and release. Journal of Nuclear Materials, 2009, 390-391, 564-567.	2.7	28
175	Erosion yields of deposited beryllium layers. Journal of Nuclear Materials, 2009, 390-391, 132-135.	2.7	27
176	The role of beryllium deuteride in plasma-beryllium interactions. Journal of Nuclear Materials, 2009, 390-391, 681-684.	2.7	68
177	An overview of a comprehensive First Mirror Test for ITER at JET. Journal of Nuclear Materials, 2009, 390-391, 1066-1069.	2.7	40
178	Codeposition of deuterium with ITER materials. Nuclear Fusion, 2009, 49, 035002.	3.5	65
179	Nanocrystalline diamond coating of fusion plasma facing components. Diamond and Related Materials, 2009, 18, 740-744.	3.9	36
180	First tests of diagnostic mirrors in a tokamak divertor: An overview of experiments in DIII-D. Fusion Engineering and Design, 2008, 83, 79-89.	1.9	37

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181	Characterization of magnetron sputtered rhodium films for reflective coatings. Surface and Coatings Technology, 2008, 202, 2837-2843.	4.8	50
182	Characterization of sub-stoichiometric rhodium oxide deposited by magnetron sputtering. Surface Science, 2008, 602, 3375-3380.	1.9	29
183	Substrate-dependent deposition efficiency on mirrors exposed in the TCV divertor. Fusion Engineering and Design, 2008, 83, 30-38.	1.9	12
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