

Chen-Yang Lu

List of Publications by Year in descending order

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56
papers

3,774
citations

201674

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docs citations

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times ranked

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citing authors

#	ARTICLE	IF	CITATIONS
1	Enhancing radiation tolerance by controlling defect mobility and migration pathways in multicomponent single-phase alloys. <i>Nature Communications</i> , 2016, 7, 13564.	12.8	533
2	Influence of chemical disorder on energy dissipation and defect evolution in concentrated solid solution alloys. <i>Nature Communications</i> , 2015, 6, 8736.	12.8	477
3	Mechanism of Radiation Damage Reduction in Equiatomic Multicomponent Single Phase Alloys. <i>Physical Review Letters</i> , 2016, 116, 135504.	7.8	359
4	A promising new class of irradiation tolerant materials: Ti ₂ ZrHfV _{0.5} Mo _{0.2} high-entropy alloy. <i>Journal of Materials Science and Technology</i> , 2019, 35, 369-373.	10.7	266
5	Effects of compositional complexity on the ion-irradiation induced swelling and hardening in Ni-containing equiatomic alloys. <i>Scripta Materialia</i> , 2016, 119, 65-70.	5.2	244
6	Radiation-induced segregation on defect clusters in single-phase concentrated solid-solution alloys. <i>Acta Materialia</i> , 2017, 127, 98-107.	7.9	212
7	Direct Observation of Defect Range and Evolution in Ion-Irradiated Single Crystalline Ni and Ni Binary Alloys. <i>Scientific Reports</i> , 2016, 6, 19994.	3.3	131
8	Point defect evolution in Ni, NiFe and NiCr alloys from atomistic simulations and irradiation experiments. <i>Acta Materialia</i> , 2015, 99, 69-76.	7.9	120
9	Influence of chemical disorder on energy dissipation and defect evolution in advanced alloys. <i>Journal of Materials Research</i> , 2016, 31, 2363-2375.	2.6	110
10	Shockwave generates 100 dislocation loops in bcc iron. <i>Nature Communications</i> , 2018, 9, 4880.	12.8	106
11	Influence of irradiation temperature on void swelling in NiCoFeCrMn and NiCoFeCrPd. <i>Scripta Materialia</i> , 2019, 158, 57-61.	5.2	74
12	Effects of Fe concentration on the ion-irradiation induced defect evolution and hardening in Ni-Fe solid solution alloys. <i>Acta Materialia</i> , 2016, 121, 365-373.	7.9	64
13	Features of primary damage by high energy displacement cascades in concentrated Ni-based alloys. <i>Journal of Applied Physics</i> , 2016, 119, .	2.5	59
14	Effect of alloying elements on defect evolution in Ni-20X binary alloys. <i>Acta Materialia</i> , 2018, 151, 159-168.	7.9	55
15	Microstructure of a 14Cr-ODS ferritic steel before and after helium ion implantation. <i>Journal of Nuclear Materials</i> , 2014, 455, 366-370.	2.7	53
16	CD13/Aminopeptidase N Is a Potential Therapeutic Target for Inflammatory Disorders. <i>Journal of Immunology</i> , 2020, 204, 3-11.	0.8	48
17	Enhanced void swelling in NiCoFeCrPd high-entropy alloy by indentation-induced dislocations. <i>Materials Research Letters</i> , 2018, 6, 584-591.	8.7	46
18	Distinct point defect behaviours in body-centered cubic medium-entropy alloy NbZrTi induced by severe lattice distortion. <i>Acta Materialia</i> , 2022, 229, 117806.	7.9	44

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19	The effect of injected interstitials on void formation in self-ion irradiated nickel containing concentrated solid solution alloys. <i>Journal of Nuclear Materials</i> , 2017, 488, 328-337.	2.7	43
20	Effect of spark plasma sintering temperature on microstructure and mechanical properties of 14Cr-ODS ferritic steels. <i>Materials Science & Engineering A: Structural Materials: Properties, Microstructure and Processing</i> , 2016, 660, 52-60.	5.6	42
21	Formation and growth of stacking fault tetrahedra in Ni via vacancy aggregation mechanism. <i>Scripta Materialia</i> , 2016, 114, 137-141.	5.2	42
22	Disorder in Mn ₁ AX _n phases at the atomic scale. <i>Nature Communications</i> , 2019, 10, 622.	12.8	41
23	Microstructure of HIPed and SPSed 9Cr-ODS steel and its effect on helium bubble formation. <i>Journal of Nuclear Materials</i> , 2016, 474, 65-75.	2.7	39
24	Enhanced Radiation-tolerant Oxide Dispersion Strengthened Steel and its Microstructure Evolution under Helium-implantation and Heavy-ion Irradiation. <i>Scientific Reports</i> , 2017, 7, 40343.	3.3	34
25	Interstitial migration behavior and defect evolution in ion irradiated pure nickel and Ni-xFe binary alloys. <i>Journal of Nuclear Materials</i> , 2018, 509, 237-244.	2.7	34
26	Effects of ion irradiation on chromium coatings of various thicknesses on a zirconium alloy. <i>Journal of Nuclear Materials</i> , 2019, 526, 151740.	2.7	34
27	Microstructures and mechanical properties of 9Cr oxide dispersion strengthened steel produced by spark plasma sintering. <i>Fusion Engineering and Design</i> , 2017, 115, 67-73.	1.9	31
28	Effects of Y ₂ O ₃ , La ₂ O ₃ and CeO ₂ additions on microstructure and mechanical properties of 14Cr-ODS ferrite alloys produced by spark plasma sintering. <i>Fusion Engineering and Design</i> , 2017, 121, 159-166.	1.9	30
29	Chemically-biased diffusion and segregation impede void growth in irradiated Ni-Fe alloys. <i>Current Opinion in Solid State and Materials Science</i> , 2019, 23, 92-100.	11.5	27
30	Microstructure of nano-structured ODS CLAM steel by mechanical alloying and hot isostatic pressing. <i>Journal of Nuclear Materials</i> , 2013, 442, S148-S152.	2.7	26
31	Investigation of defect clusters in ion-irradiated Ni and NiCo using diffuse X-ray scattering and electron microscopy. <i>Journal of Nuclear Materials</i> , 2016, 469, 153-161.	2.7	26
32	Effect of Y/Ti atomic ratio on microstructure of oxide dispersion strengthened alloys. <i>Materials Characterization</i> , 2017, 134, 35-40.	4.4	26
33	The effect of interstitial carbon atoms on defect evolution in high entropy alloys under helium irradiation. <i>Acta Materialia</i> , 2022, 233, 117955.	7.9	26
34	Effects of mechanical alloying time on microstructure and properties of 9Cr-ODS steels. <i>Journal of Nuclear Materials</i> , 2014, 455, 554-560.	2.7	25
35	Irradiation effects of medium-entropy alloy NiCoCr with and without pre-indentation. <i>Journal of Nuclear Materials</i> , 2019, 524, 60-66.	2.7	25
36	Current development of body-centered cubic high-entropy alloys for nuclear applications. <i>Tungsten</i> , 2021, 3, 197-217.	4.8	24

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37	Radiation-assisted chemical short-range order formation in high-entropy alloys. <i>Scripta Materialia</i> , 2022, 212, 114547.	5.2	24
38	From suppressed void growth to significant void swelling in NiCoFeCr complex concentrated solid-solution alloy. <i>Materialia</i> , 2020, 9, 100603.	2.7	22
39	Elucidating He-H assisted cavity evolution in alpha Cr under multiple ion beam irradiation. <i>Scripta Materialia</i> , 2020, 187, 291-295.	5.2	18
40	Tunable mechanical property and strain hardening behavior of a single-phase CoFeNi ₂ V _{0.5} Mo _{0.2} high entropy alloy. <i>Materials Science & Engineering A: Structural Materials: Properties, Microstructure and Processing</i> , 2020, 776, 139027.	5.6	16
41	Crossover from disordered to core-shell structures of nano-oxide Y ₂ O ₃ dispersed particles in Fe. <i>Applied Physics Letters</i> , 2016, 109, .	3.3	15
42	TiN films fabricated by reactive gas pulse sputtering: A hybrid design of multilayered and compositionally graded structures. <i>Applied Surface Science</i> , 2016, 389, 255-259.	6.1	13
43	Current state and prospect on the development of advanced nuclear fuel system materials: A review. <i>Materials Reports Energy</i> , 2021, 1, 100007.	3.2	12
44	Angiogenic and Arthritogenic Properties of the Soluble Form of CD13. <i>Journal of Immunology</i> , 2019, 203, 360-369.	0.8	11
45	High radiation tolerance of an ultrastrong nanostructured NiCoCr alloy with stable dispersed nanooxides and fine grain structure. <i>Journal of Nuclear Materials</i> , 2021, 557, 153316.	2.7	11
46	Indentation behaviour of ion-irradiated X-750 Ni-based superalloy. <i>Philosophical Magazine Letters</i> , 2017, 97, 101-109.	1.2	10
47	Improved irradiation tolerance of reactive gas pulse sputtered TiN coatings with a hybrid architecture of multilayered and compositionally graded structures. <i>Journal of Nuclear Materials</i> , 2018, 501, 388-397.	2.7	8
48	Multi-axial and multi-energy channeling study of disorder evolution in ion-irradiated nickel. <i>Journal of Nuclear Materials</i> , 2019, 525, 92-101.	2.7	8
49	PREPARATION AND CHARACTERIZATION OF NANO-STRUCTURED 14Cr-ODS FERRITIC STEEL. <i>Jinshu Xuebao/Acta Metallurgica Sinica</i> , 2012, 48, 649.	0.3	8
50	Improved irradiation tolerance of W thin films with homogeneously multilayered structure. <i>Surface and Coatings Technology</i> , 2017, 313, 230-235.	4.8	7
51	Highly stable nanocrystalline oxide dispersion strengthened alloys with outstanding helium bubble suppression. <i>Journal of Nuclear Materials</i> , 2021, 557, 153283.	2.7	7
52	Soluble CD13 induces inflammatory arthritis by activating the bradykinin receptor B1. <i>Journal of Clinical Investigation</i> , 2022, 132, .	8.2	6
53	Preface to the special issue on high entropy materials and tungsten-based nuclear materials. <i>Tungsten</i> , 2021, 3, 117-118.	4.8	1
54	Effect of Carbon on Dislocation Loops Formation during Self-Ion Irradiation in Fe-Cr Alloys at High Temperatures. <i>Materials</i> , 2022, 15, 2211.	2.9	1

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55	Influence of Milling Time on Morphology and Properties of Precursor Powders for 9Cr Oxide Dispersion Strengthened Steel. <i>Advanced Materials Research</i> , 2014, 887-888, 219-222.	0.3	0
56	Formation of Bubble-Loop Complexes During Helium Radiation in Fe-9Cr Steel. <i>Frontiers in Energy Research</i> , 2021, 9, .	2.3	0