Jerome Lamontagne

List of Publications by Year
in descending order

24 papers	1,148 citations	759233 12 h-index	642732 23 g-index
24 all docs	24 docs citations	24 times ranked	929 citing authors

#	Article	IF	Citations
1	Comparison by Fourier transform infrared (FTIR) spectroscopy of different ageing techniques: application to road bitumens. Fuel, 2001, 80, 483-488.	6.4	488
2	Infrared microscopy investigation of oxidation and phase evolution in bitumen modified with polymers. Fuel, 2008, 87, 1270-1280.	6.4	175
3	Detailed characterisations of high burn-up structures in oxide fuels. Journal of Nuclear Materials, 2008, 372, 318-339.	2.7	140
4	FTIR and SUVF spectroscopy applied to reservoir compartmentalization: a comparative study with gas chromatography fingerprints results. Fuel, 2002, 81, 861-866.	6.4	66
5	Direct and continuous methodological approach to study the ageing of fossil organic material by infrared microspectrometry imaging: application to polymer modified bitumen. Analytica Chimica Acta, 2001, 444, 241-250.	5.4	53
6	HIGH BURNUP CHANGES IN UO₂FUELS IRRADIATED UP TO 83 GWD/T IN M5^(R)CLADDINGS. Nuclear Engineering and Technology, 2009, 41, 155-162.	2.3	32
7	Fission Gas Bubbles Characterisation in Irradiated UO2 Fuel by SEM, EPMA and SIMS. Mikrochimica Acta, 2006, 155, 183-187.	5.0	25
8	Fission products and nuclear fuel behaviour under severe accident conditions part 2: Fuel behaviour in the VERDON-1 sample. Journal of Nuclear Materials, 2017, 495, 49-57.	2.7	21
9	Detection of Gas Bubble by SIMS in Irradiated Nuclear Fuel. Mikrochimica Acta, 2004, 145, 91-94.	5.0	20
10	A method for the quantification of total xenon concentration in irradiated nuclear fuel with SIMS and EPMA. Nuclear Instruments & Methods in Physics Research B, 2008, 266, 147-154.	1.4	18
11	Microbeam Analysis of Irradiated Materials: Practical Aspects. Microscopy and Microanalysis, 2007, 13, 150-155.	0.4	16
12	Evidence of tellurium iodide compounds in a power-ramped irradiated UO2 fuel rod. Journal of Nuclear Materials, 2013, 437, 409-414.	2.7	14
13	Heterogeneous UO2 fuel irradiated up to a high burn-up: Investigation of the HBS and of fission product releases. Journal of Nuclear Materials, 2013, 442, 309-319.	2.7	12
14	Swelling under irradiation of MgO pellets containing americium oxide: The ECRIX-H irradiation experiment. Journal of Nuclear Materials, 2011, 413, 137-144.	2.7	11
15	Experimental evidence of oxygen thermo-migration in PWR UO2 fuels during power ramps using in-situ oxido-reduction indicators. Journal of Nuclear Materials, 2016, 480, 32-39.	2.7	10
16	ECRIX-H experiment: Synthesis of post-irradiation examinations and simulations. Journal of Nuclear Materials, 2011, 415, 158-166.	2.7	8
17	Annealing tests of in-pile irradiated oxide coated Uâ€“Mo/Alâ€“Si dispersed nuclear fuel. Journal of Nuclear Materials, 2014, 452, 533-547.	2.7	8
18	Evidence of a Biphasic Domain in the UO₂-Nd₂O₃Diagram at Room Temperature: a Proof for a Miscibility Gap in UO₂-Nd₂O₃ Phase Diagram ?. Solid State Phenomena, 0, 172-174, 624-629.	0.3	7

#	Article	IF	Citations
19	Determining the americium transmutation rate and fission rate by post-irradiation examination within the scope of the ECRIX-H experiment. Journal of Nuclear Materials, 2013, 440, 366-376.	2.7	7
20	Study of structural material resulting from the nuclear fuel cycle using SEM-WDX, EPMA and SIMS techniques. Mikrochimica Acta, 2008, 161, 355-362.	5.0	5
21	Americium, curium and neodymium analysis in ECRIX-H irradiated pellet: sample preparation for TIMS measurements. Radiochimica Acta, 2013, 101, 293-300.	1.2	4
22	Experimental evidence of the formation of a new chemical phase in a power ramped UO2 nuclear fuel. Journal of Nuclear Materials, 2015, 457, 246-251.	2.7	3
23	Fission products behaviour during a power transient: Their inventory in an intragranular bubble. Journal of Nuclear Materials, 2017, 493, 225-229.	2.7	3
24	Analysis of Pu by Virtual-standard WDS-EPMA. Results of an Interlaboratory Round-robin Test. Microscopy and Microanalysis, 2008, 14, 1094-1095.	0.4	2