Gilles Noguere

List of Publications by Year in descending order

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	567281	197818
2,558	15	49
citations	h-index	g-index
63	63	1534
docs citations	times ranked	citing authors
	citations 63	2,558 15 citations h-index 63 63

#	Article	IF	CITATIONS
1	Generation of thermal scattering files with the CINEL code. EPJ Nuclear Sciences & Technologies, 2022, 8, 8.	0.7	2
2	CONRAD â \in " a code for nuclear data modeling and evaluation. EPJ Nuclear Sciences & Technologies, 2021, 7, 10.	0.7	9
3	Atomic scale Monte-Carlo simulations of neutron diffraction experiments on stoichiometric uranium dioxide up to 1664 K. Nuclear Instruments and Methods in Physics Research, Section A: Accelerators, Spectrometers, Detectors and Associated Equipment, 2021, 1002, 165251.	1.6	3
4	Temperature-dependent dynamic structure factors for liquid water inferred from inelastic neutron scattering measurements. Journal of Chemical Physics, 2021, 155, 024502.	3.0	7
5	Non-destructive analysis of samples with a complex geometry by NRTA. Journal of Analytical Atomic Spectrometry, 2020, 35, 478-488.	3.0	4
6	HPRL – International cooperation to identify and monitor priority nuclear data needs for nuclear applications. EPJ Web of Conferences, 2020, 239, 15005.	0.3	15
7	The joint evaluated fission and fusion nuclear data library, JEFF-3.3. European Physical Journal A, 2020, 56, 1.	2.5	318
8	Average neutron cross sections of <mml:math xmlns:mml="http://www.w3.org/1998/Math/MathML"><mml:mmultiscripts><mml:mi mathvariant="normal">Tc</mml:mi><mml:mprescripts></mml:mprescripts><mml:none></mml:none><mml:mrow></mml:mrow></mml:mmultiscripts></mml:math> . Physical Review	2.9	6
9	C, 2020, 102,. Combining density functional theory and Monte Carlo neutron transport calculations to study the phonon density of states of UO2 up to 1675ÅK by inelastic neutron scattering. Physical Review B, 2020, 102,.	3.2	3
10	Evaluation of neutron induced reactions on 56Fe with CONRAD. EPJ Web of Conferences, 2020, 239, 11005.	0.3	5
11	Neutron resonance transmission analysis of cylindrical samples used for reactivity worth measurements. Journal of Radioanalytical and Nuclear Chemistry, 2019, 321, 519-530.	1.5	4
12	107Ag and 109Ag resonance parameters for neutron induced reactions below 1â€keV. Nuclear Instruments & Methods in Physics Research B, 2019, 446, 19-28.	1.4	9
13	-wave average neutron resonance parameters of <mml:math xmlns:mml="http://www.w3.org/1998/Math/MathML"><mml:mrow><mml:mmultiscripts><mml:mi>Lu</mml:mi> /><mml:none></mml:none><mml:mn>175</mml:mn></mml:mmultiscripts><mml:mo>+</mml:mo><mml:mi>n</mml:mi></mml:mrow></mml:math>	2.9	7
14	Physical Review C. 2019, 100, . Systematics of Nd cumulative fission yields for neutron-induced fission of 235U, 238U, 238Pu, 239Pu, 240Pu and 241Pu. European Physical Journal Plus, 2018, 133, 1.	2.6	2
15	IAEA CIELO Evaluation of Neutron-induced Reactions on 235 U and 238 U Targets. Nuclear Data Sheets, 2018, 148, 254-292.	2.2	33
16	ENDF/B-VIII.0: The 8 th Major Release of the Nuclear Reaction Data Library with CIELO-project Cross Sections, New Standards and Thermal Scattering Data. Nuclear Data Sheets, 2018, 148, 1-142.	2.2	1,324
17	CIELO Collaboration Summary Results: International Evaluations of Neutron Reactions on Uranium, Plutonium, Iron, Oxygen and Hydrogen. Nuclear Data Sheets, 2018, 148, 189-213.	2.2	73
18	Evaluation of Neutron-induced Cross Sections and their Related Covariances with Physical Constraints. Nuclear Data Sheets, 2018, 148, 383-419.	2.2	12

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19	Generation of the ^{1 < /sup>H in H _{2 < /sub>O neutron thermal scattering law covariance matrix of the CAB model. EPJ Nuclear Sciences & Technologies, 2018, 4, 32.}}	0.7	6
20	Nuclear data adjustment based on the interpretation of post-irradiation experiments with the DARWIN2.3 package. EPJ Nuclear Sciences & Technologies, 2018, 4, 47.	0.7	4
21	Evaluation of the Neutron Data Standards. Nuclear Data Sheets, 2018, 148, 143-188.	2.2	159
22	Doppler broadening of neutron-induced resonances using ab initio phonon spectrum. European Physical Journal Plus, 2018, 133, 1.	2.6	5
23	Covariance matrices of the hydrogen neutron cross sections bound in light water for the JEFF-3.1.1 neutron library. Annals of Nuclear Energy, 2017, 104, 132-145.	1.8	9
24	Improving nuclear data accuracy of 241Am and 237Np capture cross sections. EPJ Web of Conferences, 2017, 146, 11035.	0.3	3
25	Measurement of double differential cross-section of light water at high temperature and pressure to generate $S(\hat{l}\pm,\hat{l}^2)$. EPJ Web of Conferences, 2017, 146, 13006.	0.3	5
26	Impact of the thermal scattering law of H in H ₂ O on the isothermal temperature reactivity coefficients for UOX and MOX fuel lattices in cold operating conditions. EPJ Nuclear Sciences & Technologies, 2016, 2, 28.	0.7	12
27	Generation of 238U Covariance Matrices by Using the Integral Data Assimilation Technique of the CONRAD Code. EPJ Web of Conferences, 2016, 106, 04015.	0.3	7
28	Resonance parameter and covariance evaluation for ¹⁶ O up to 6 MeV. EPJ Nuclear Sciences & Technologies, 2016, 2, 43.	0.7	14
29	Measurements of the effective cumulative fission yields of 143Nd, 145Nd, 146Nd, 148Nd and 150Nd for 235U in the PHENIX fast reactor. EPJ Nuclear Sciences & Technologies, 2016, 2, 32.	0.7	4
30	The Use of Nuclear Data as Nuisance Parameters in the Integral Data Assimilation of the PROFIL Experiments. Nuclear Science and Engineering, 2016, 182, 377-393.	1.1	5
31	Improved Mixed Oxide Fuel Calculations with the Evaluated Nuclear Data Library JEFF-3.2. Nuclear Science and Engineering, 2016, 182, 135-150.	1.1	2
32	Partial-wave analysis ofn+Am241 reaction cross sections in the resonance region. Physical Review C, 2015, 92, .	2.9	10
33	Evaluation of Cross Section Uncertainties Using Physical Constraints: Focus on Integral Experiments. Nuclear Data Sheets, 2015, 123, 178-184.	2.2	18
34	Feedback on ²³⁹ Pu and ²⁴⁰ Pu nuclear data and associated covariances through the CERES integral experiments. Journal of Nuclear Science and Technology, 2015, 52, 1044-1052.	1.3	2
35	Zero Variance Penalty Model for the Generation of Covariance Matrices in Integral Data Assimilation Problems. Nuclear Science and Engineering, 2012, 172, 164-179.	1.1	15
36	Interpretation of Fission Product Oscillations in the MINERVE Reactor, from Thermal to Epithermal Spectra. Nuclear Science and Engineering, 2011, 169, 229-244.	1.1	21

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37	Average radiation widths of levels in natural xenon isotopes. Nuclear Physics A, 2011, 870-871, 131-158.	1.5	12
38	Interpretation of pile-oscillation measurements by the integral data assimilation technique. Nuclear Instruments and Methods in Physics Research, Section A: Accelerators, Spectrometers, Detectors and Associated Equipment, 2011, 629, 288-295.	1.6	9
39	Fission Product Cross Section Evaluations using Integral Experiments. Journal of the Korean Physical Society, 2011, 59, 1343-1346.	0.7	4
40	New ⁵⁶ Fe Covariances for the JEFF3 File from the Feedback of Integral Benchmark Analysis. Nuclear Science and Engineering, 2010, 166, 267-275.	1.1	7
41	Retroactive Generation of Covariance Matrix of Nuclear Model Parameters Using Marginalization Techniques. Nuclear Science and Engineering, 2010, 166, 276-287.	1.1	31
42	Interpretation of PERLE Experiment for the Validation of Iron Nuclear Data Using Monte Carlo Calculations. Nuclear Science and Engineering, 2010, 166, 89-106.	1.1	19
43	Neutron average cross sections of Np237. Physical Review C, 2010, 81, .	2.9	6
44	Average neutron parameters for hafnium. Nuclear Physics A, 2009, 831, 106-136.	1.5	13
45	Group-average covariance matrices for the hafnium isotopes of interest for light water reactor applications. Annals of Nuclear Energy, 2009, 36, 1059-1069.	1.8	1
46	A Monte Carlo Approach to Nuclear Model Parameter Uncertainties Propagation. Nuclear Science and Engineering, 2009, 161, 363-370.	1.1	27
47	Generalization of the SPRT Method for the Modeling of the Neutron Cross Sections in the Unresolved Resonance Range. Nuclear Science and Engineering, 2009, 162, 76-86.	1.1	11
48	Modeling of the <i>n </i> + ²⁴² Pu Reactions for Fast Reactor Applications. Nuclear Science and Engineering, 2009, 162, 178-191.	1.1	10
49	A nuclear data oriented interface code for processing applications. Annals of Nuclear Energy, 2008, 35, 2259-2269.	1.8	7
50	Assessment and Propagation of the 237Np Nuclear Data Uncertainties in Integral Calculations by Monte Carlo Techniques. Nuclear Science and Engineering, 2008, 160, 108-122.	1.1	19
51	Analysis of the PROFIL and PROFIL-2 Sample Irradiation Experiments in Phénix for JEFF-3.1 Nuclear Data Validation. Nuclear Science and Engineering, 2008, 160, 232-241.	1.1	25
52	Neutron capture and total cross sections of 1127 and 1129. Physical Review C, 2006, 74, .	2.9	25
53	Experimental Tests of the Crystal Lattice Model of the R-Matrix Code SAMMY. AIP Conference Proceedings, 2005, , .	0.4	3
54	The Resolution Function in Neutron Time-of-Flight Experiments. Journal of Nuclear Science and Technology, 2002, 39, 685-688.	1.3	12