

Volker Metz

List of Publications by Year in descending order

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Version: 2024-02-01

31
papers

721
citations

623734

14
h-index

580821

25
g-index

31
all docs

31
docs citations

31
times ranked

738
citing authors

#	ARTICLE	IF	CITATIONS
1	Monte-Carlo based investigation of individual dosimetry in deep geological repository for high-level nuclear waste with consideration of realistic body postures. <i>Annals of Nuclear Energy</i> , 2021, 161, 108414.	1.8	1
2	Fifteen Years of Radionuclide Research at the KIT Synchrotron Source in the Context of the Nuclear Waste Disposal Safety Case. <i>Geosciences (Switzerland)</i> , 2019, 9, 91.	2.2	19
3	Barite recrystallization in the presence of ²²⁶ Ra and ¹³³ Ba. <i>Geochimica Et Cosmochimica Acta</i> , 2018, 232, 124-139.	3.9	31
4	Comparison of calculated and measured radionuclide inventory of a Zircaloy-4 cladding tube plenum section. <i>MRS Advances</i> , 2018, 3, 1031-1037.	0.9	3
5	Mineralogical characterization of scalings formed in geothermal sites in the Upper Rhine Graben before and after the application of sulfate inhibitors. <i>Geothermics</i> , 2018, 71, 264-273.	3.4	15
6	Summary of the Euratom Collaborative Project FIRST-Nuclides and Conclusions for the Safety Case. <i>Nuclear Technology</i> , 2017, 198, 260-276.	1.2	4
7	Interdisziplinäre Analysen von Entsorgungsoptionen für radioaktive Reststoffe – der Beitrag geochemisch-basierter Analysen. <i>Energie in Naturwissenschaft, Technik, Wirtschaft Und Gesellschaft</i> , 2016, , 17-23.	0.0	0
8	Study of the release of the fission gases (Xe and Kr) and the fission products (Cs and I) under anoxic conditions in bicarbonate water. <i>Materials Research Society Symposia Proceedings</i> , 2015, 1744, 35-41.	0.1	2
9	Physico-chemical characterization of a spent UO ₂ fuel with respect to its stability under final disposal conditions. <i>Materials Research Society Symposia Proceedings</i> , 2014, 1665, 283-289.	0.1	3
10	Adsorption of dissolved aluminum on sapphire-c and kaolinite: implications for points of zero charge of clay minerals. <i>Geochemical Transactions</i> , 2014, 15, 9.	0.7	29
11	Nucleation and growth kinetics of RaxBa _{1-x} SO ₄ solid solution in NaCl aqueous solutions. <i>Geochimica Et Cosmochimica Acta</i> , 2014, 125, 290-307.	3.9	24
12	Radium removal in a large scale evaporitic system. <i>Geochimica Et Cosmochimica Acta</i> , 2013, 103, 121-137.	3.9	20
13	Net surface proton excess of smectites obtained from a combination of potentiometric acid-base, mass and electrolyte titrations. <i>Colloids and Surfaces A: Physicochemical and Engineering Aspects</i> , 2012, 412, 11-19.	4.7	7
14	Co-precipitation of radium in high ionic strength systems: 1. Thermodynamic properties of the Na-Ra-Cl-SO ₄ -H ₂ O system – Estimating Pitzer parameters for RaCl ₂ . <i>Geochimica Et Cosmochimica Acta</i> , 2011, 75, 5389-5402.	3.9	36
15	Co-precipitation of radium in high ionic strength systems: 2. Kinetic and ionic strength effects. <i>Geochimica Et Cosmochimica Acta</i> , 2011, 75, 5403-5422.	3.9	35
16	Formation of (Ba,Ra)SO ₄ Solid Solutions – Results from Barite (Re)Precipitation and Coprecipitation Experiments. <i>Springer Geology</i> , 2011, , 635-642.	0.3	0
17	Chemical status of U(VI) in cemented waste forms under saline conditions. <i>Radiochimica Acta</i> , 2010, 98, 674-683.	1.2	11
18	Modelling Long-Term Corrosion of Cemented Waste Forms in Salt Brines. , 2009, , .		0

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19	Effects of hydrogen and bromide on the corrosion of spent nuclear fuel and ^{137}Cs -irradiated $\text{UO}_2(\text{s})$ in NaCl brine. <i>Radiochimica Acta</i> , 2008, 96, 637-648.	1.2	14
20	Geochemically Based Safety Assessment. <i>Journal of Nuclear Science and Technology</i> , 2007, 44, 470-476.	1.3	11
21	Geochemically Based Safety Assessment. <i>Journal of Nuclear Science and Technology</i> , 2007, 44, 470-476.	1.3	4
22	Alteration Behavior of High Burnup Spent Fuel in Salt Brine Under Hydrogen Overpressure and in Presence of Bromide. <i>Materials Research Society Symposia Proceedings</i> , 2006, 985, 1.	0.1	0
23	Radionuclide release from high burnup spent fuel during corrosion in salt brine in the presence of hydrogen overpressure. <i>Journal of Nuclear Materials</i> , 2005, 346, 24-31.	2.7	31
24	Stoichiometry of smectite dissolution reaction. <i>Geochimica Et Cosmochimica Acta</i> , 2005, 69, 1755-1772.	3.9	91
25	Towards the establishment of a reliable proxy for the reactive surface area of smectite. <i>Geochimica Et Cosmochimica Acta</i> , 2005, 69, 2581-2591.	3.9	78
26	Geochemically derived non-gaseous radionuclide source term for the Asse salt mine "assessment for the use of a $\text{Mg}(\text{OH})_2$ -based backfill material. <i>Radiochimica Acta</i> , 2004, 92, 819-825.	1.2	6
27	Surface protonation data of kaolinite "reevaluation based on dissolution experiments. <i>Journal of Colloid and Interface Science</i> , 2003, 264, 67-75.	9.4	46
28	Radionuclide Source Term for the ASSE Salt Mine: Geochemical Assessment for the Use of Magnesium(II) Based Backfill Material. , 2003, , .		0
29	Site Specific Sorption Data for the Asse Salt Mine. , 2003, , .		0
30	The effect of pH and temperature on kaolinite dissolution rate under acidic conditions. <i>Geochimica Et Cosmochimica Acta</i> , 2002, 66, 3913-3926.	3.9	123
31	Stirring effect on kaolinite dissolution rate. <i>Geochimica Et Cosmochimica Acta</i> , 2001, 65, 3475-3490.	3.9	77