

Balazs Fekete

List of Publications by Year in descending order

Source: <https://exaly.com/author-pdf/12156625/publications.pdf>

Version: 2024-02-01

11
papers

112
citations

1478505

6
h-index

1372567

10
g-index

11
all docs

11
docs citations

11
times ranked

86
citing authors

#	ARTICLE	IF	CITATIONS
1	Prediction of corrosion fatigue crack growth rate in alloys. Part I: General corrosion fatigue model for aero-space aluminum alloys. Corrosion Science, 2018, 141, 22-29.	6.6	22
2	The electrochemical properties of alloy 690 in simulated pressurized water reactor primary water: Effect of temperature. Journal of Nuclear Materials, 2019, 518, 305-315.	2.7	20
3	Prediction of corrosion fatigue crack growth rate in alloys. Part II: effect of electrochemical potential, NaCl concentration, and temperature on crack propagation in AA2024-T351. Corrosion Science, 2019, 152, 130-139.	6.6	18
4	New energy-based low cycle fatigue model for reactor steels. Materials & Design, 2015, 79, 42-52.	5.1	16
5	Customization of the coupled environment fracture model for predicting stress corrosion cracking in Alloy 600 in PWR environment. Corrosion Science, 2018, 139, 58-67.	6.6	12
6	Isothermal and thermal-mechanical fatigue of VVER-440 reactor pressure vessel steels. Journal of Nuclear Materials, 2015, 464, 394-404.	2.7	11
7	An advanced coupled environment fracture model for hydrogen-induced cracking in alloy 600 in PWR primary heat transport environment. Theoretical and Applied Fracture Mechanics, 2018, 95, 233-241.	4.7	6
8	Low cycle thermomechanical fatigue of VVER-440 reactor pressure vessel steels: Investigation the fatigue kinetics and development of a life assessment model. Procedia Structural Integrity, 2016, 2, 2164-2172.	0.8	3
9	Low Cycle Fatigue Behavior of VVER-440 Reactor Pressure Vessel Steels at Isothermal Condition. Materials Science Forum, 2015, 812, 47-52.	0.3	2
10	Acoustic Barkhausen Effect Observed in Various Steels. Materials Science Forum, 0, 885, 208-215.	0.3	1
11	Lifetime estimation of a BWR core shroud in terms of IGSCC. Nuclear Engineering and Design, 2020, 368, 110831.	1.7	1